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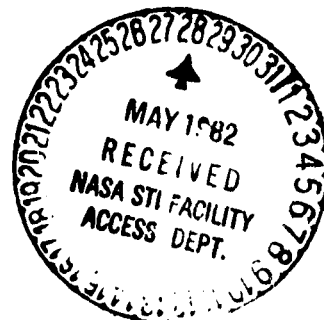
# **STUDY OF REACTOR BRAYTON POWER SYSTEMS FOR NUCLEAR ELECTRIC SPACECRAFT**

**FOR**

**CALIFORNIA INSTITUTE OF TECHNOLOGY  
JET PROPULSION LABORATORY**

**CONTRACT 955008**

**SEPTEMBER 28, 1979**



**AIRESEARCH MANUFACTURING COMPANY OF ARIZONA**

**A Division of The Garrett Corporation  
Phoenix, Arizona**

**31-3321**



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PHOENIX, ARIZONA

TECHNICAL REPORT  
STUDY OF REACTOR BRAYTON POWER SYSTEMS  
FOR NUCLEAR ELECTRIC SPACECRAFT

31-3321

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Initial Issue

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LASL Reactor Data

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## SUMMARY

The study of Brayton power systems for nuclear electric spacecraft was performed to provide a basis for comparison between this system and others that have been under study for some time. Most significantly, this initial study has yielded performance parameters for the Brayton system that are very competitive with the alternative systems and envelope dimensions that are compatible with the Space Shuttle payload bay.

The primary performance parameters of system mass and radiator area were determined for systems from 100 to 1000 kW<sub>e</sub>. Mathematical models of all system components were used to determine masses and volumes. Two completely independent systems provide propulsion power so that no single-point failure can jeopardize a mission. The waste heat radiators utilize armored heat pipes to limit meteorite puncture. The armor thickness was statistically determined to achieve the required probability of survival.

A 400-kW<sub>e</sub> reference system received primary attention as required by the contract. The components of this system were defined and a conceptual layout was developed with encouraging results. An arrangement with redundant 400-kW<sub>e</sub> Brayton power systems having a 1500°K (2240°F) turbine inlet temperature (TIT) was shown to be compatible with the dimensions of the Space Shuttle orbiter payload bay. The spacecraft is deployed from within the cylindrical primary radiator in a manner similar to the present Jet Propulsion Laboratory (JPL) thermionic system design. The preliminary mass determination for the complete power system is close to the desired 20 kg/kW<sub>e</sub> for the specified Jovian environment. With further refinement, that the current Brayton conceptual design can better this goal. Study results have also shown that use of more advanced technology (higher TIT) will substantially improve system performance characteristics.



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Because certain near-term missions with nuclear electric power systems are of present interest, a preliminary design concept of a 100-kW<sub>e</sub> Brayton system was also developed. This system was designed with essentially current Brayton technology (i.e., TIT = 1325°K) for operation in the geostationary orbital environment. A flight version of this system could be available by the late 1980s.

Further studies and analyses of refined nuclear reactor Brayton space power systems are recommended for continued attention. Brayton system technology efforts should be undertaken in the near future to assure a proper base for development of flight systems in the late 1980s and 1990s.

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PHOENIX, ARIZONA**ACKNOWLEDGEMENTS**

The excellent working relationship which has been established with JPL has contributed substantially to this study. Especially recognized are the efforts of Wayne Phillios, who was technical manager of the study, Jack Mondt, Leader of the Nuclear Thermal to Electric Power Group, and Teh Hsieh, who monitored technical aspects throughout this study. The support of James Lazar and Jerome Mullin of the NASA Office of Aeronautics and Space Technology Space Power and Propulsion Division is greatly appreciated. The continuing interest of Mr. Robert English of the NASA Lewis Research Center has been very helpful and is acknowledged with much appreciation.

In support of this study, JPL arranged for data to be provided by the Los Alamos Scientific Laboratory (Dave Buden, Dan Koerig and Ken Cooper) and by Thermacore, Inc. (Yale Eastman and Don Ernst). The extensive dialog and numerous meetings with the above named persons and others at these organizations have greatly enhanced this study.



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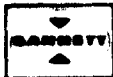


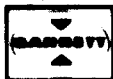
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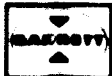
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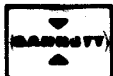




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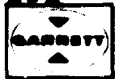
## STUDY OF REACTOR BRAYTON POWER SYSTEMS FOR NUCLEAR ELECTRIC SPACECRAFT

### 1.0 INTRODUCTION

Studies are currently underway at the Jet Propulsion Laboratory (JPL) and at the Los Alamos Scientific Laboratory (LASL) to demonstrate the technical feasibility of nuclear reactor-powered spacecraft propelled by electric rocket thrusters. Such vehicles would be capable of performing detailed explorations of the solar system including rapid trips to the outer planets with massive payloads during the 1990s and into the 21st century. Particular emphasis has been placed on the definition of and on technology development for the power conversion subsystem with thermionics currently receiving primary attention.

The purpose of this study is to provide comparative information on an alternative conversion system, the Closed Brayton power system, to allow meaningful comparisons.

As a result of the large technological data base available from the development of numerous gas turbines as well as the significant R&D funding on Closed Brayton Cycle (CBC) engines over the past ten years, such engines should be considered an available technology with excellent potential for future development. Major questions to be resolved include definition of optimum operating parameters and effective integration with spacecraft elements including the launch vehicle, the nuclear subsystem (reactor and shield), the waste heat rejection system, and the space science payload. These topics are addressed in the three tasks that have been defined for this study, as described in Section 2.0. Estimation of system reliability and lifetime characteristics is of great interest, but could only be addressed



in a very preliminary way within the confines of this study. The remainder of this introduction provides additional definition of the requirements for the Nuclear Electric Spacecraft.

### 1.1 Study Background

Since the 1950s, the limitations of chemical rockets for extensive exploration of the solar system and other high-energy missions have been well understood. Electric-rocket propulsion was given considerable analytical and development attention during the past two decades and space flight tests were conducted. In comparison studies, nuclear electric rocket propulsion was repeatedly shown to be performance and economically effective for the more demanding missions when they would be flown.

In the early 1970s, the post-Apollo emphasis on Earth applications and, especially, the development of the Space Shuttle transportation system caused space program planners to terminate all work on nuclear systems except for radioisotope thermoelectric generators (RTGs), some advanced reactor concepts, and compatible power conversion research.

During the past several years, projections of future space mission requirements, both military and civil, have resulted in a renewed interest in advanced nuclear systems. The Department of Energy (DOE) has technology development programs in advanced nuclear energy sources for space at LASL. The NASA Office of Aeronautics and Space Technology (OAST) Space Power and Electric Propulsion Division is funding power conversion technology developments, primarily in thermionics and thermoelectrics. Although Brayton power conversion in space has a history of over 20 years, the present study represents the first effort in recent years to assess the component and system aspects in some detail using both available and obtainable technology. In particular, this study addresses the critical issue of space environment



compatible radiator design which has been recognized for some time as the major drawback to application of CBC technology.

## 1.2 Missions for Nuclear Electric Rocket Propelled Spacecraft

The primary missions identified by JPL for this study are solar system exploration of the outer planets with massive payloads. Figure 1 shows the net\* 400-kW<sub>e</sub> nuclear electric, rocket propelled spacecraft mass versus time of flight for various thruster exhaust velocities. Two cases are shown with different masses of shielding for the nuclear reactor. The required shielding mass is probably between these cases. Performance is also shown for a 60-kW<sub>e</sub> solar-electric rocket-propelled spacecraft. As can be seen, substantial masses can be placed in Jupiter orbit(1)\*\* which would represent a likely first use for the nuclear electric spacecraft. Reference 1 also contains data on flights to other outer planets and on a solar escape mission with high net mass and reasonable trip times. Further trajectory optimization of these and other high-energy missions are expected to result in a strong recommendation for the development of nuclear electric rocket propulsion capability.

In a more recent paper(2), Phillips and Pawlik discuss the design of a nuclear electric propulsion system, including the selection of thrusters and propellant for outer planet (Saturn, Uranus and Neptune) exploration. A power level between 200 and 250 kW<sub>e</sub> is recommended with current technology for the early missions and growth potential for more difficult later missions. These missions could be accommodated without significant changes in the basic nuclear reactor heat source and heat rejection system.

\*Total spacecraft mass less the following term for Cases A and B--(1.03 propulsion system mass plus shielding mass).

\*\*Numbers in parenthesis refer to the list of references in Section 6.0.

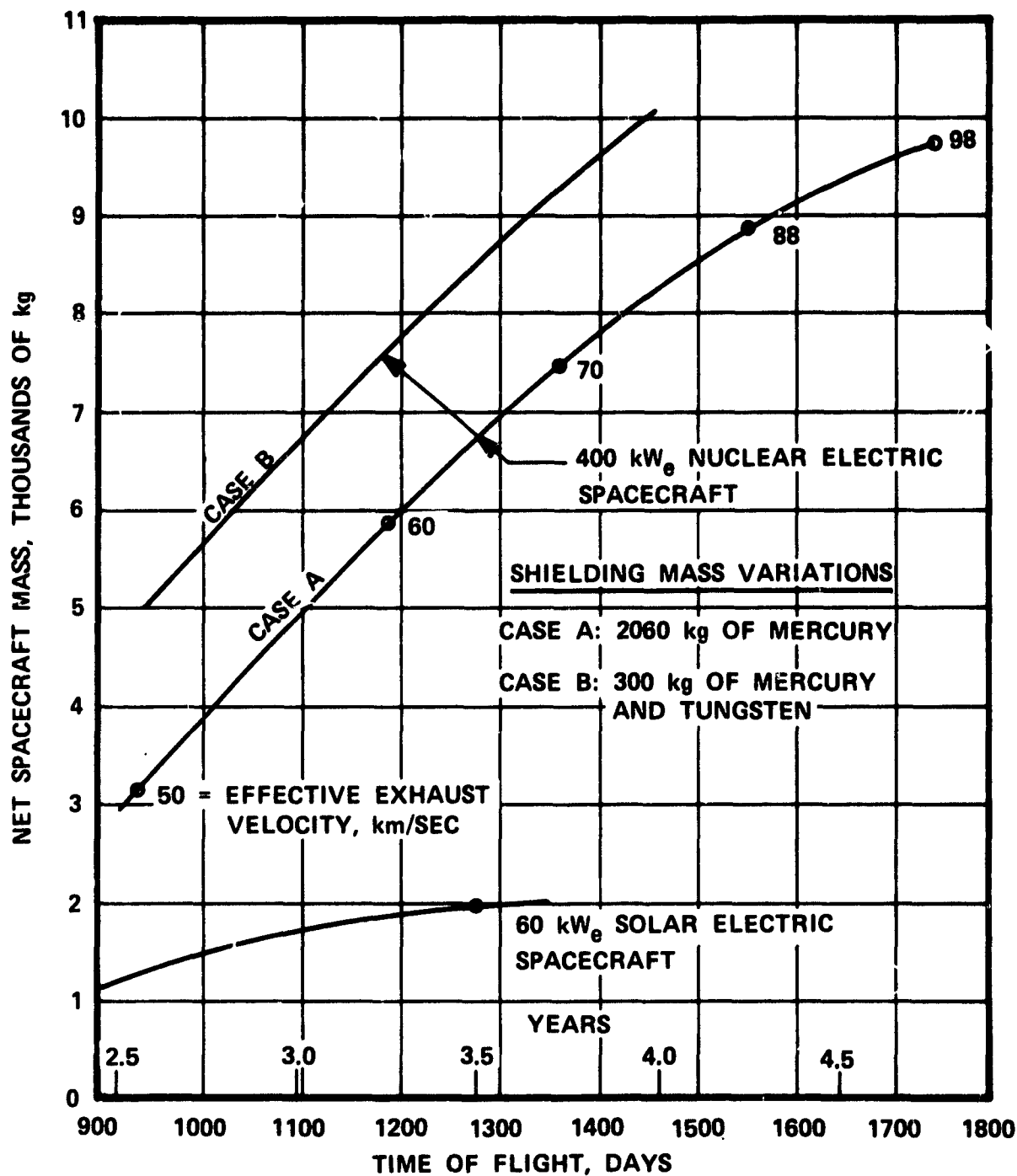


Figure 1. Net Spacecraft Mass in Jupiter Orbit Vs Time of Flight.



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Other high-energy missions in the 1990s and beyond will include such candidates as orbital transfer vehicles, both in geocentric orbits and throughout cislunar space, highly maneuverable military spacecraft, and the initial versions of solar system cruisers.

### 1.3 Overall Study Approach

The study of CBC systems depends on the use of computer-based analytical methods. Only with such methods can thousands of candidate systems be designed and evaluated parametrically. A computer model based on appropriate guidelines and constraints has been created showing the variables intrinsic to CBC systems to be studied. The guidelines for the model are discussed in Section 1.4. The computer study method is described in Section 2.1.1.

A reference system design at 400 kW<sub>e</sub> was evaluated in substantial detail and is discussed at some length in Section 2.1.2. Because of prospective interest in lower power systems, a preliminary layout of a 100-kW<sub>e</sub> system was prepared and is shown in Section 2.1.3. Analytical results for systems from 100 to 1000 kW<sub>e</sub> using near-term and obtainable Brayton technologies are given in Section 2.1.4.

The major technical challenge of this study was the identification of a credible heat-pipe radiator. Early in the study, the Brayton power conversion system design parameters were selected from computer results which did not include all system components and which included a liquid-cooled radiator. A heat-pipe radiator was then designed using a separate radiator computer program and the mass adjusted accordingly. In the latter phase of the study, these computer models were merged and analytical representations of all system components were included. Achievement of this overall design tool is a major accomplishment of this study. Throughout this investigation, layouts of radiators were made and evaluated against two constraints--reasonable mass and ability to fit into the Space Shuttle bay.



Specific heat-pipe designs were evaluated by Thermacore, Inc. under a JPL contract. The study results are discussed in Section 2.2.4 and presented in toto in Appendix A. The radiator geometry is discussed in greater detail in Section 2.3.

#### 1.4 Study Guidelines

There were two primary goals of this study--first, to study nuclear electric propulsion (NEP) power systems from 100 to 1000 kW<sub>e</sub>, and second, to create a reference design including a system layout at 400 kW<sub>e</sub>. The following constraints and guidelines were given in the JPL contract (3) o. were expressed as highly desirable by JPL representatives:

- (a) The system should be designed for technology attainable in the 1985 to 1990 time frame.
- (b) The system must produce the voltage level desired by the ion thrusters.
- (c) System components must be placed within the shadow of the reactor shield.
- (d) The 400-kW<sub>e</sub> system and payload should fit within the Space Shuttle bay.
- (e) These systems should be designed to operate in a recently defined Jovian micrometeoroid environment throughout the mission life.
- (f) System lifetime is 120,000 hours.
- (g) The system specific mass at 400 kW<sub>e</sub> should be less than 20 kg/kW<sub>e</sub>.





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The following additional guidelines were assumed by AiResearch:

- (a) No single-point failures are allowed; hence, all systems include the mass of a redundant system.
- (b) The 100-kW<sub>e</sub> system would use essentially state of the art technology.
- (c) The 400- and 1000-kW<sub>e</sub> systems can utilize longer term technology.
- (d) Turbomachinery design is current state of the art.



## 2.0 TECHNICAL DISCUSSION

### 2.1 Task 1 - Power System Conceptual Design Studies

The method and the results of the study of power systems between 100 and 1000 kW<sub>e</sub> are described in the following sections. Conceptual designs at 400 and 100 kW<sub>e</sub> are presented in greater detail.

#### 2.1.1 Study Method

The CBC computer design program was used to design thousands of candidate systems. This program requires input of key thermodynamic parameters to begin the cycle design. The output is complete preliminary design of the resultant systems, including masses of all components in both tabular and plotted forms. These systems are examined in detail and a reference system is selected. From the geometry of the components, a layout of the selected design can be made.

Table 1 lists the array of parameters that were studied. The most significant parameter is power level because system mass is a strong function of power level. The TIT and cycle temperature ratio\* are the next most important parameters. The cycle temperature ratio dictates the cycle efficiency (from the Carnot efficiency equation) depending upon the other parameters selected. TIT coupled with the cycle temperature ratio affects radiator size. The compressor specific speed, rotor speed, and power level determine the performance level of the turbomachinery and system pressure level. Rotor speed also determines the mass of the turbomachinery. Recuperator effectiveness affects thermal input, the mass of the recuperator, and radiator size. The pressure loss parameter has a strong effect on the

\*Ratio of compressor inlet temperature (CIT) to TIT.



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**TABLE 1**  
**INITIAL BRAYTON POWER SYSTEM STUDY PARAMETERS**

	<u>Units</u>	<u>Values</u>			
Net Output Power Level	kW <sub>e</sub>	100	400	1000	
Turbine Inlet Temperature	°K	1150	1325	1500	1650
Cycle Temperature Ratio (CIT/TIT)		0.25 to 0.40			
Compressor Specific Speed		0.07 to 0.15			
Rotating Speed	krpm	12 to 48			
Recuperator Effectiveness		0.88 to 0.97			
Pressure Loss Parameter		0.92			



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mass of the heat exchangers and system performance. The value that was selected (0.92) allows good system performance and reasonable heat exchanger mass.

Other variables that affect system mass include compressor pressure ratio, heat exchanger pressure drop, radiator sink temperature, and meteoroid flux. Compressor pressure ratio is selected to yield maximum system performance. Heat exchanger pressure drops are split between the heat exchangers for minimum system mass. Reactor thermal power affects both reactor and shield mass. The meteoroid flux can drastically affect radiator mass. All of these parameters are modeled in the computer code.

The computer program is a series of overlay programs, i.e., all of the thermodynamic cycles are defined first; then, the recuperator for each cycle is designed, next the radiator for each cycle is designed, etc. Finally, these results are matched, listed, and plotted. Figure 2 shows the manner in which information is transferred between these programs.

The first and most important step is the cycle analysis. This program uses the input parameters shown in Table 1. Some other secondary inputs, such as alternator design characteristics, are also used. This program, based on empirical performance maps of the compressor and turbine, will design a thermodynamic cycle for a given combination of the above parameters. Among the information defined is compressor and turbine efficiencies and sizes, alternator windage and size, cycle efficiency, and the thermodynamic state points. From the thermodynamic state point and rotor size, the rotating unit mass is calculated. This design point information is stored on the computer disk. The program continues to design cycles until all combinations of parameters are exhausted.



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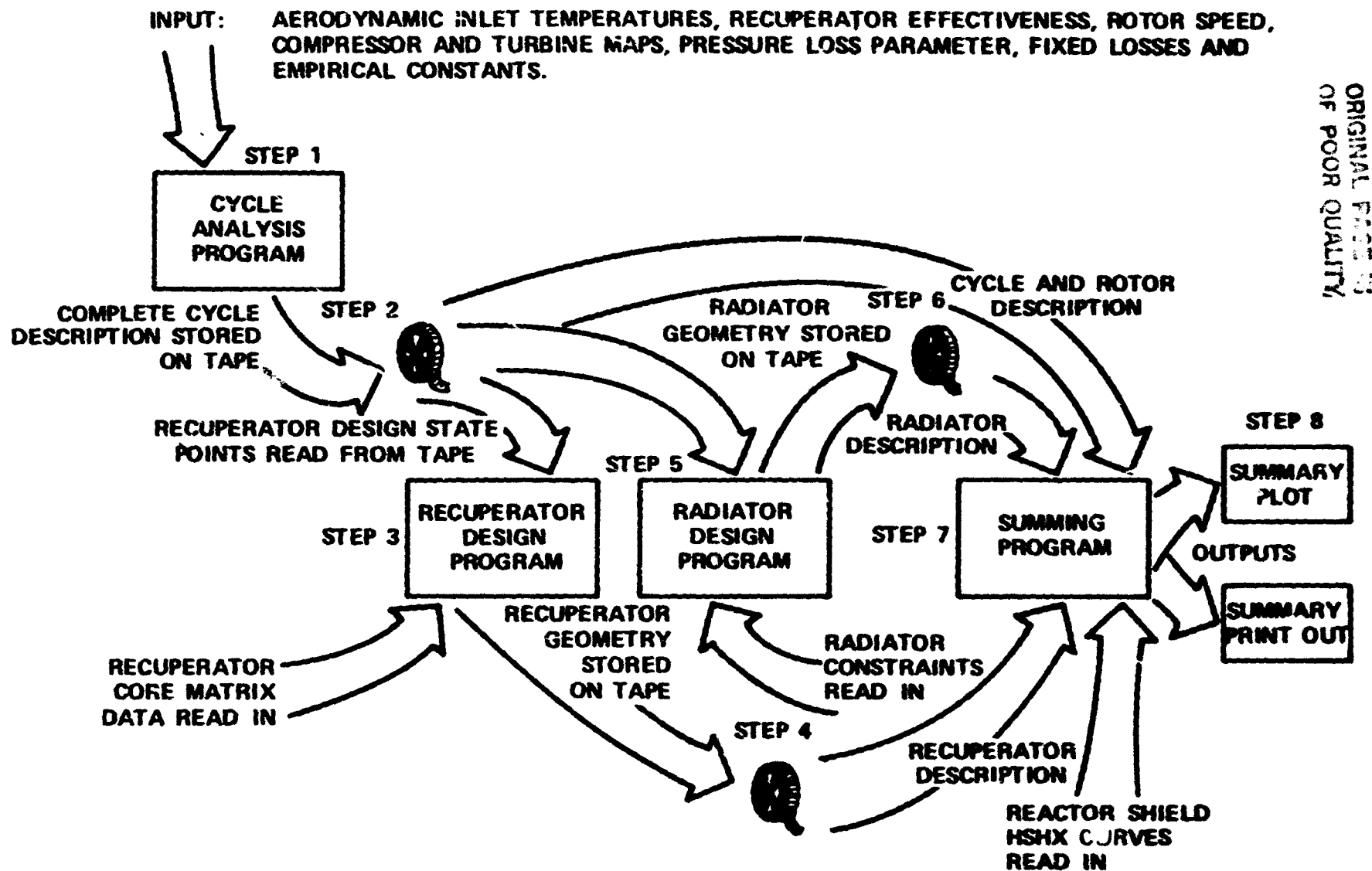


Figure 2. Brayton Space Power System Design Methodology

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The next step is the design of the recuperator. The recuperator core matrix and other minor inputs are read in. Next, the cycle state point data are read from the computer disk. The recuperator is designed, and its mass and geometry are stored on the computer disk. After all of the recuperators are designed, the radiator design begins.

The heat-pipe radiator design program was added as a program option during this study. The calculation method is discussed in Section 2.3.6 (Radiator Conceptual Design). Like the recuperator program, it requires some basic input such as heat-pipe diameter, spacing, and length; gas heat exchanger data; micrometeoroid model; etc. It also uses the cycle state point data to design a heat-pipe radiator. The dimensions and mass of each radiator are stored on the computer disk. After a heat pipe radiator has been designed for each cycle, the calculation proceeds to the summing program.

In the summing program, the information stored on the computer disk is merged to form a complete system description. The mass of the remaining system components is calculated as a function of the appropriate cycle variables. Examples are duct mass, reactor mass, shield mass, and insulation mass. After the mass of every component is defined, the total system mass, system specific mass, and specific radiator area are calculated. An abbreviated cycle description is printed, and the specific mass and area are plotted. The plots are shown and discussed in Section 2.1.4. When a candidate system is selected, an option that prints the geometry and performance of all of the components is used. From this geometry, a system layout can be made.

#### 2.1.2 Reference System Design at 400 kW<sub>e</sub>

The 400-kW<sub>e</sub> system is illustrated in Figure 3. This configuration uses technology expected to be available at least by 1990 and is



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JOVIAN ENVIRONMENT TIT =  $1500^{\circ}\text{K}$  ( $2240^{\circ}\text{F}$ )

### MASS SUMMARY

	kg
REACTOR	850
REACTOR SHIELD	800
HEAT SOURCE HEAT EXCHANGERS (2)	420
CRU (2)	430
ALTERNATOR RADIATOR	150
RECUPERATORS (2)	730
RADIATOR	4040
DUCTING & MISCELLANEOUS	400
POWER CONDITIONING	150
STRUCTURE	300
	<hr/> 8270

SPECIFIC MASS =  $20.7 \text{ kg/kW}_e$

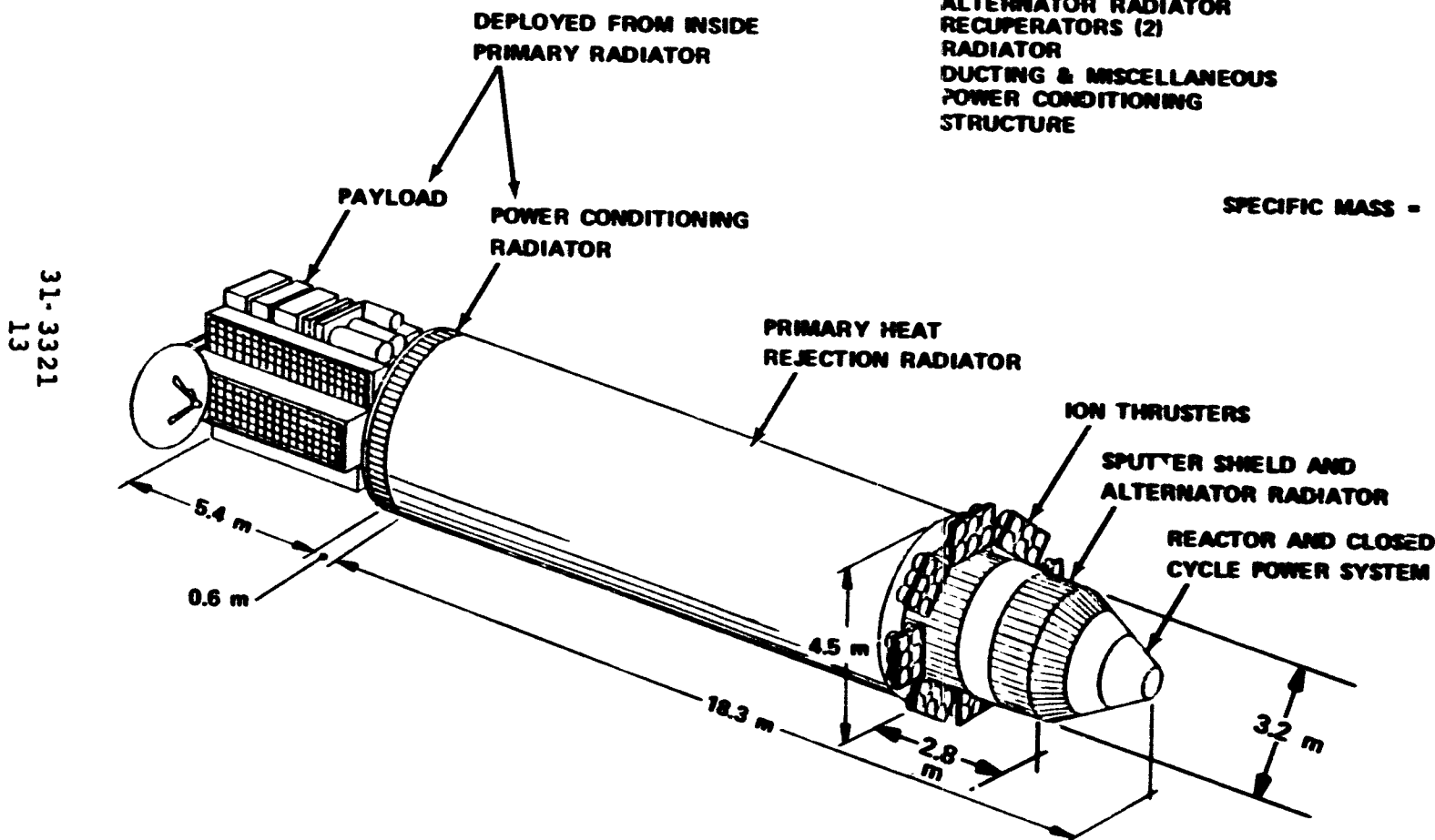


Figure 3. Nuclear Electric Spacecraft Design with a  $400\text{-kW}_e$  Brayton Power System

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designed for operation near Jupiter. The largest and most noticeable component is the primary heat rejection radiator. Because this entire system must fit into the Space Shuttle bay, the radiator length was selected with care.

The spacecraft is located forward of all other components. To fit the payload bay envelope, several components are deployed after the unit is released from the Shuttle. The spacecraft and power-conditioning radiator both telescope from inside the primary heat rejection radiator (this is also a feature of the baseline thermionic system). The ion thruster panels are rotated to a position normal to the axis of the spacecraft.

At the aft end of the spacecraft are the reactor and closed cycle power conversion systems. The reactor is the rearmost component. Inside the sputter shield and alternator radiator are the dual rotating groups, recuperators, and heat source heat exchangers. This configuration is simple and very compact.

Figure 3 shows that the specific mass is  $20.7 \text{ kg/kW}_e$ . With further refinement, the system specific mass could be decreased to meet or be less than the  $20 \text{ kg/kW}_e$  design goal. For example, relaxation of the micrometeoroid environment to reflect the relatively low fraction of the mission duration spent close to Jupiter would result in the goal being surpassed without further refinement. More complete detail on this system is given in Section 2.3.

### 2.1.3 Preliminary Conceptual Design at $100 \text{ kW}_e$

The  $100\text{-kW}_e$  conceptual design was studied in a very cursory fashion near the end of the study because of indications of early mission interest in this power level. Figure 4 shows the power system applied to a large telescoping and deploying space antenna. This





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NEAR EARTH ENVIRONMENT TIT =  $1325^{\circ}\text{K}$  ( $1925^{\circ}\text{F}$ )

MASS SUMMARY

	kg
REACTOR	350
REACTOR SHIELD	220
HEAT SOURCE HEAT EXCHANGERS (2)	120
CRU (2)	150
ALTERNATOR RADIATOR	60
RECUPERATORS (2)	350
RADIATOR	1340
STRUCTURE, DUCTING & MISCELLANEOUS	350
POWER CONDITIONING AND CONTROLS	100
	<hr/> 3040

SPECIFIC MASS =  $30.4 \text{ kg/kW}_e$

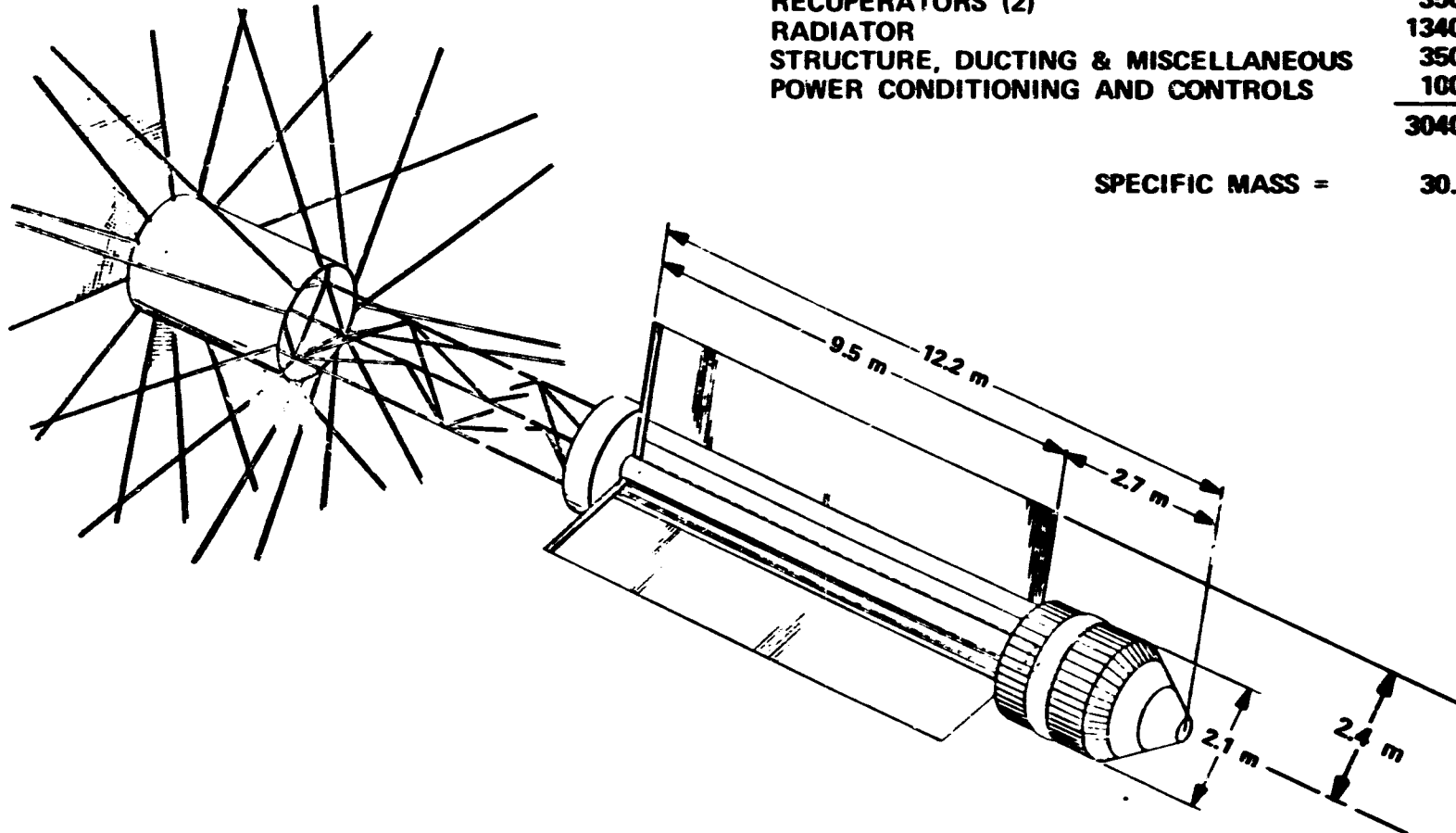


Figure 4. Preliminary  $100\text{-kW}_e$  Nuclear Electric Spacecraft Design

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power system configuration will fit easily into the Space Shuttle payload bay. The component arrangement is generally similar to the 400-kW<sub>e</sub> system described previously. The mass summary given in Figure 4 reflects a conservative design based essentially on currently available technology. Two completely independent systems provide propulsion power so that no single-point failure can jeopardize a mission. Although the radiator is "Y" shaped, a cylindrical radiator could also be used which would be more compact but more massive.

The specific mass of 30.4 kg/kW<sub>e</sub> could quite clearly be significantly reduced by the use of more advanced technology (higher TIT). Even at the lower TIT, appreciable reduction could be achieved with further refinement. Unfortunately, such refinement was not possible within the resources available for this study.

#### 2.1.4 Analytical Results from 100- to 1000-kW<sub>e</sub> Studies with Near-Term and Obtainable Brayton Technologies

Figures 5 through 12 are "shotgun" plots generated by the previously described method (Section 2.1.1). Representative plots are included for three output powers (100, 400, and 1000 kW<sub>e</sub>). These plots are presented as representative examples of the analytical approach but do not reflect a significant improvement (use of dual diameter or necked heat pipes), which was made relatively late in the study. The effects of this improvement are described later in this section. Each point on these plots represents a specific system design according to the parameters of Table 1. Each set of results consists of separate plots of specific mass and specific radiator area. The specific mass includes the heat source, two completely redundant loops for all the power conversion components, the waste heat radiator, and required power conditioning components.

Plots of the 400-kW<sub>e</sub> system, designed for a 1995 flight with a 1500°K (2240°F) TIT are shown in Figures 5 through 8. A five-year



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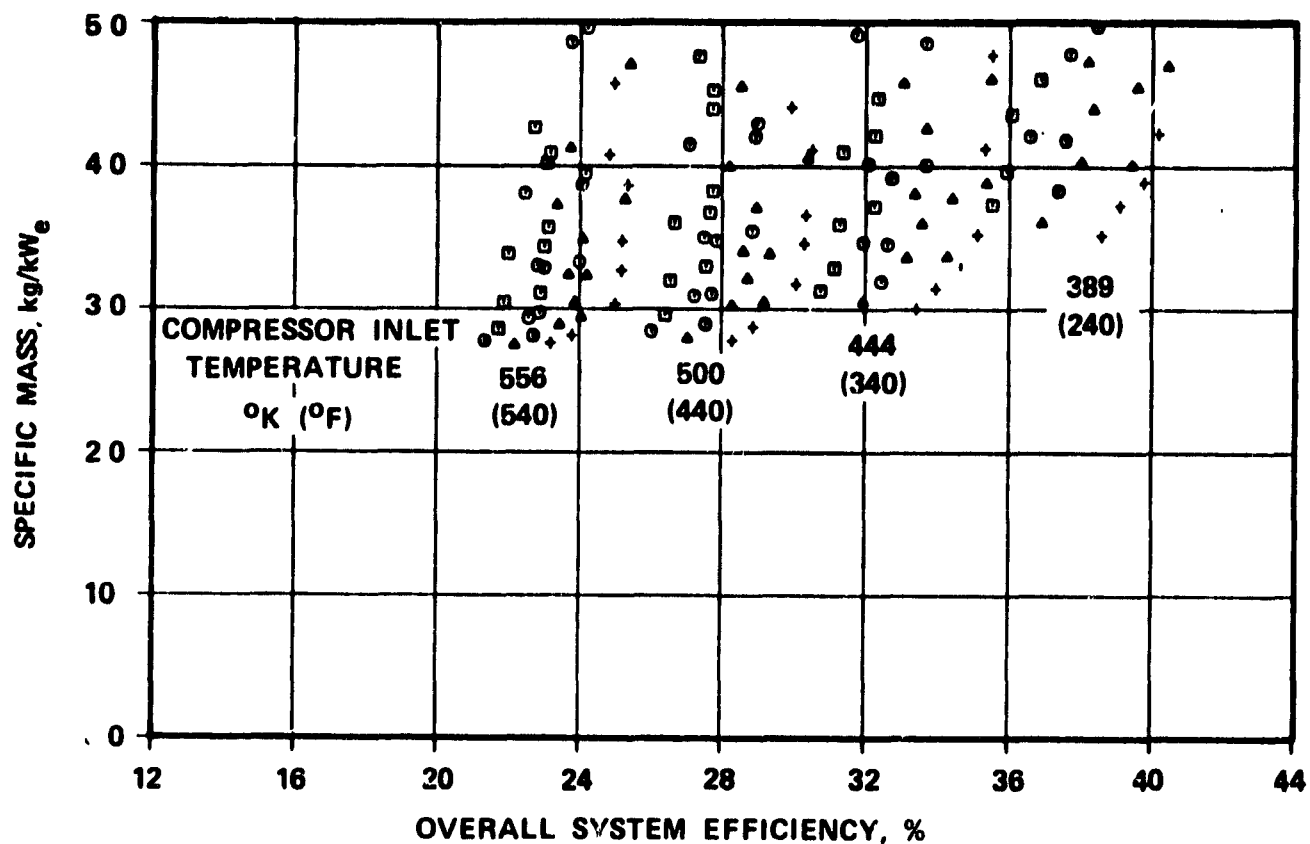


Figure 5. Specific Mass vs Overall System Efficiency of 400-kW<sub>e</sub> System at a Turbine Inlet Temperature of 1500°K (2240°F) in a Jovian Environment



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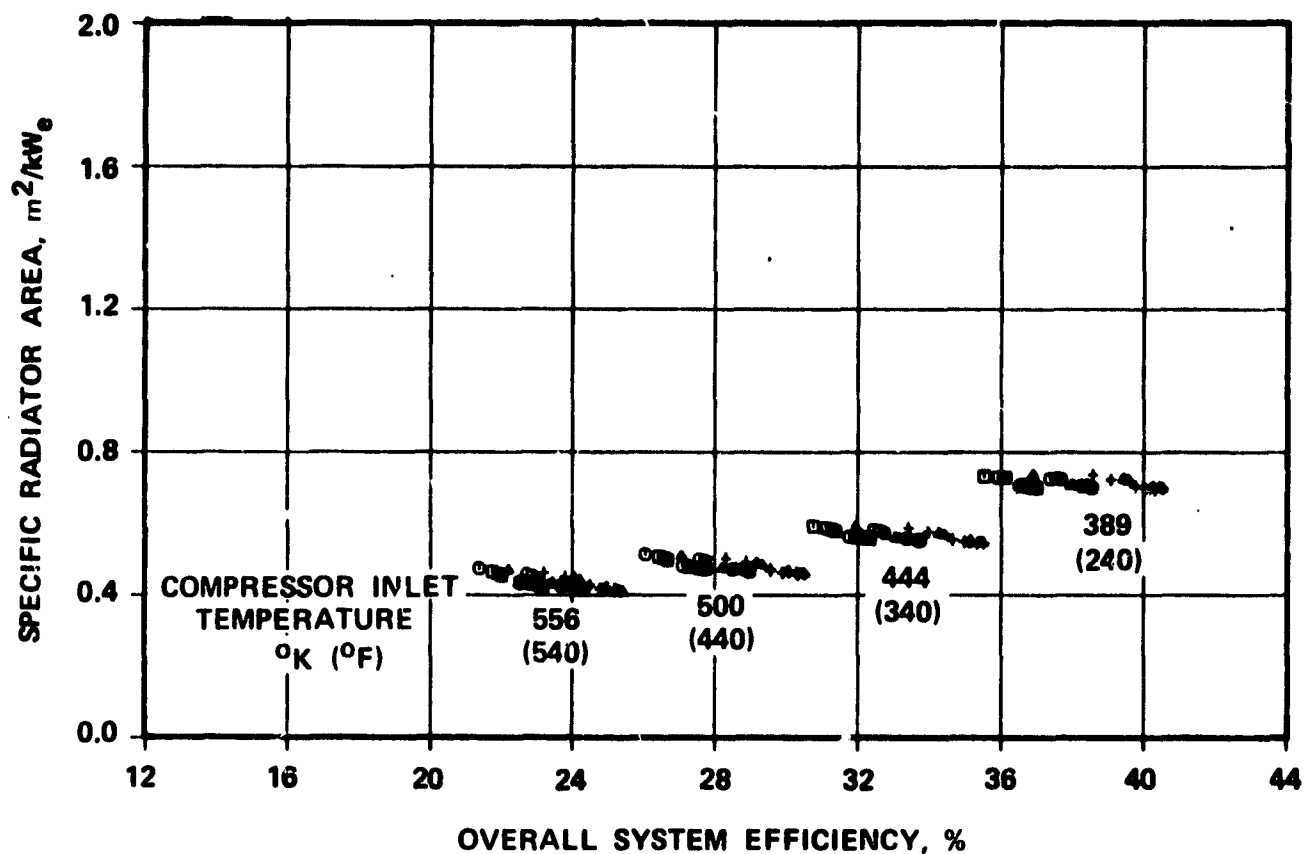


Figure 6. Specific Radiator Area vs Overall System Efficiency of 400- $kW_e$  System at a Turbine Inlet Temperature of 1500°K (2240°F) in a Jovian Environment



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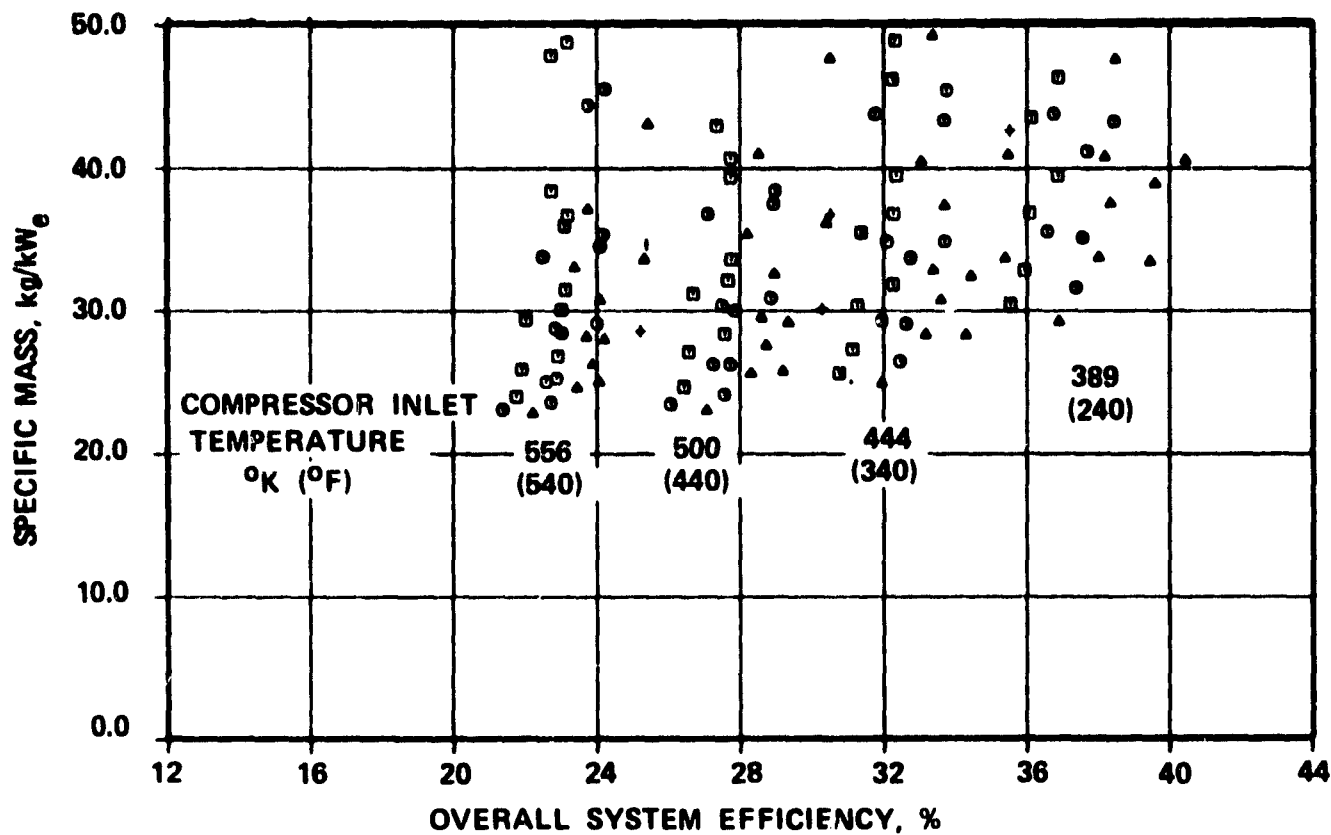


Figure 7. Specific Mass vs Overall System Efficiency of 400-kW<sub>e</sub> System at a Turbine Inlet Temperature of 1500°K (2240°F) in a Near-Earth Environment



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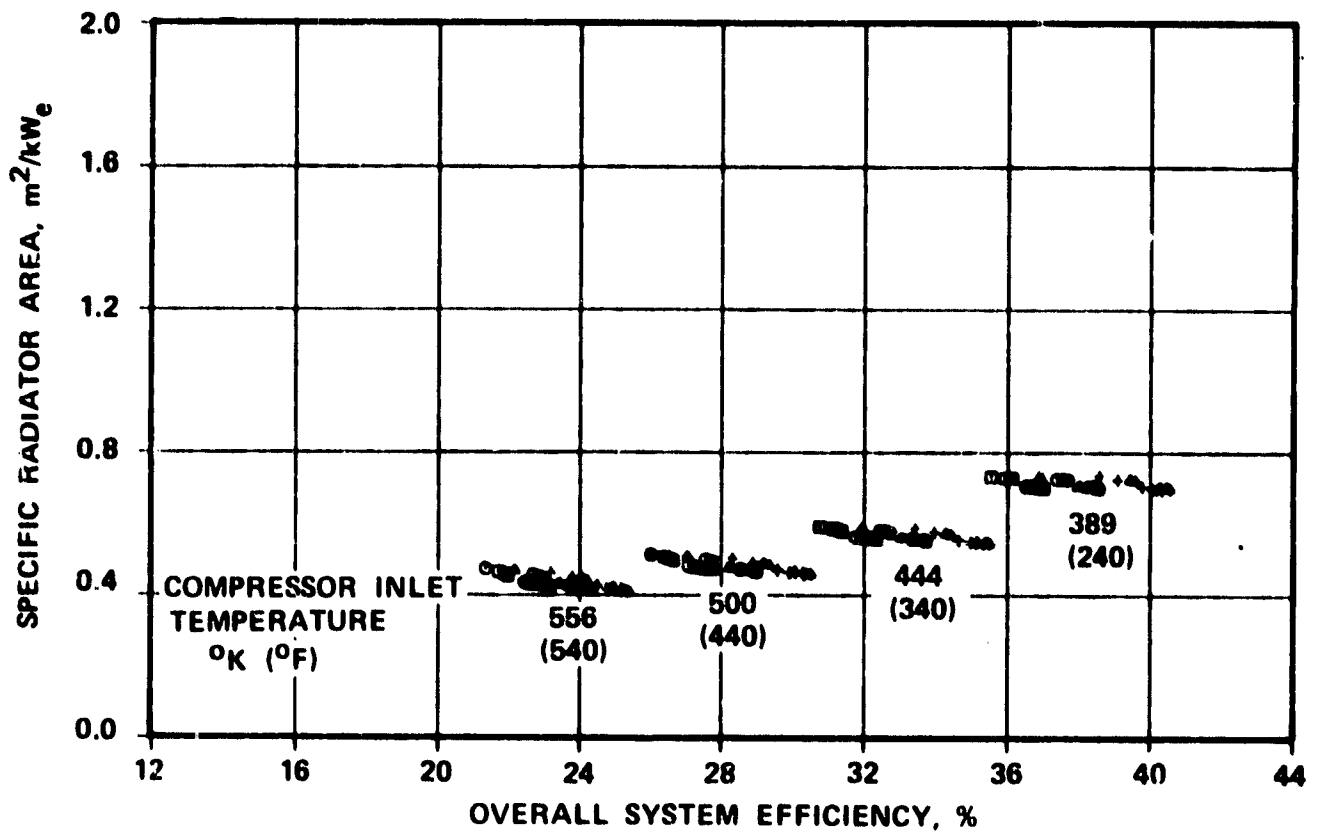


Figure 8. Specific Radiator Area vs Overall System Efficiency of 400-kW<sub>e</sub> System at a Turbine Inlet Temperature of 1500°K (2240°F) in a Near Earth-Environment



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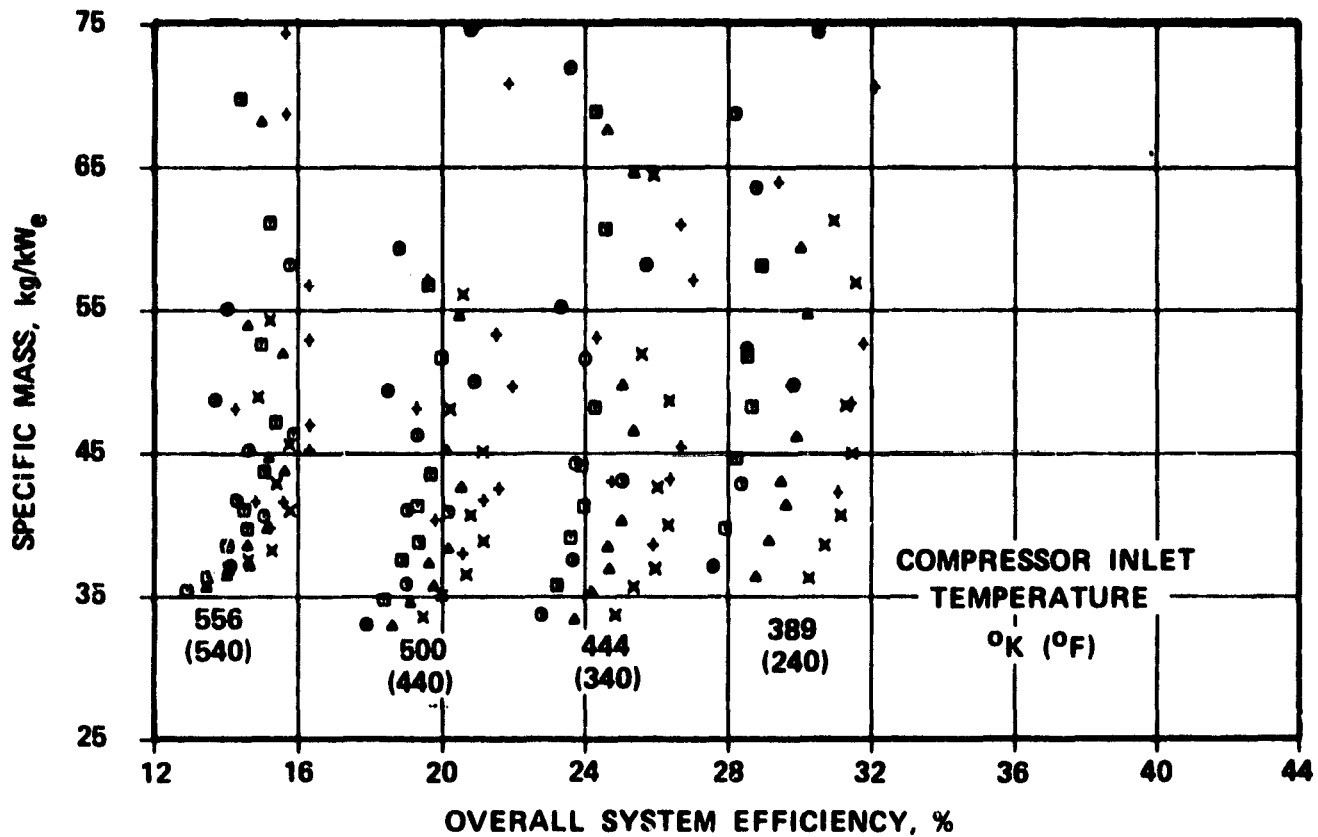


Figure 9. Specific Mass vs Overall System Efficiency of 100-kW<sub>e</sub> System at a Turbine Inlet Temperature of 1325°K (1925°F) in a Near-Earth Environment



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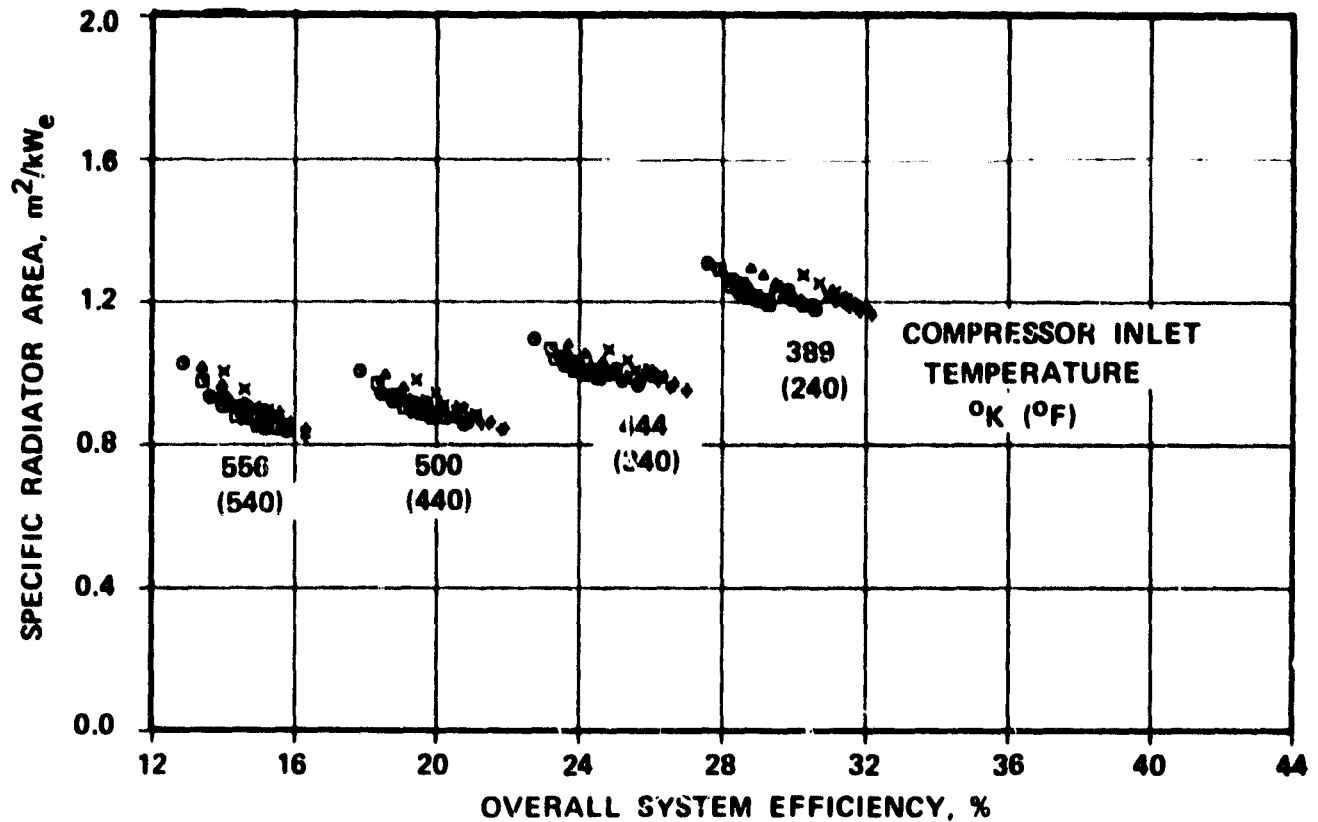


Figure 10. Specific Radiator Area vs Overall System Efficiency of 100-kW<sub>e</sub> System at a Turbine Inlet Temperature of 1325°K (1925°F) in a Near-Earth Environment





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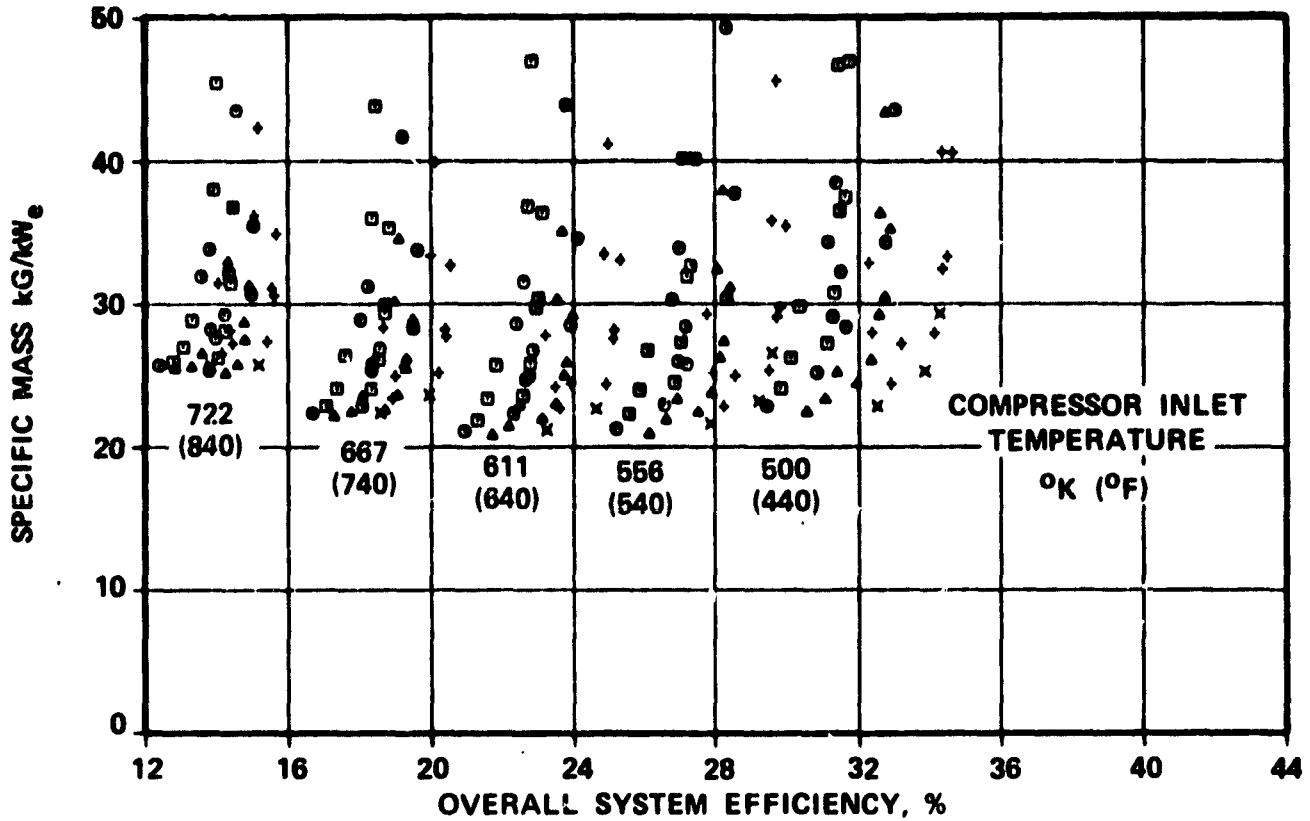
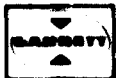


Figure 11. Specific Mass vs Overall System Efficiency of 1000-kW<sub>e</sub> System at a Turbine Inlet Temperature of 1650°K (2510°F) in a Jovian Environment



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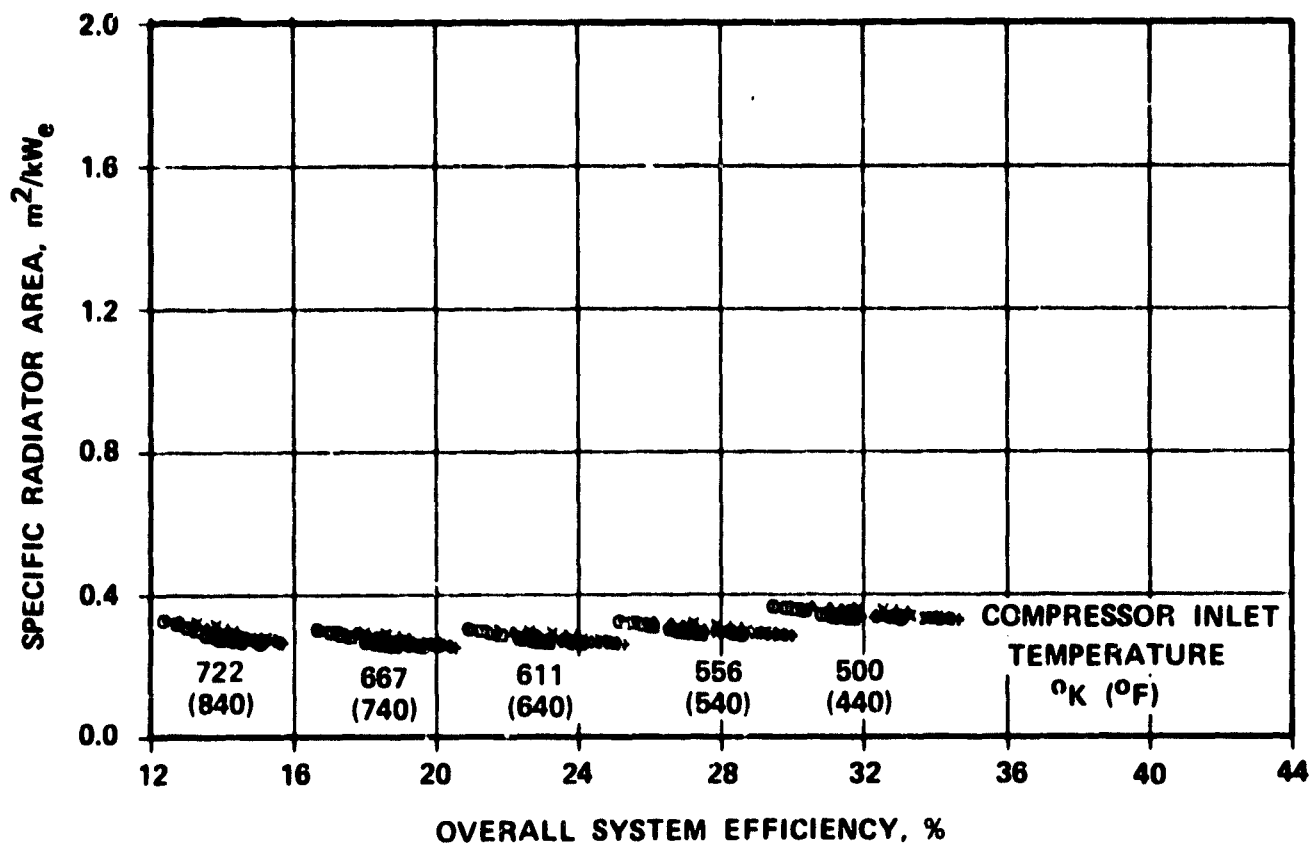


Figure 12. Specific Radiator Area vs Overall System Efficiency of 1000- $kW_e$  System at a Turbine Inlet Temperature of 1650°K (2510°F) in a Jovian Environment

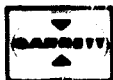


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flight system development is assumed with the result that this TIT, which can be attained with high-strength refractory materials, should be state-of-the-art by 1990. The inferences of this technology development schedule are discussed subsequently. The waste heat radiators for these systems employ the fixed cylindrical geometry shown in Figure 3. Previously, a hinged, three-panel design had been considered that was somewhat less massive but required in-space assembly operations (automated welding or brazing at the 400-kW<sub>e</sub> power level. Comparison of the specific mass data for the Jovian (Figure 5) and near-Earth (Figure 7) environments shows the substantial effect of the meteoroid armor requirement. These systems have masses that are well within the capability of a single Shuttle launch and dimensions that fit within the payload bay.

Figures 9 and 10 show the specific mass and specific radiator area for 100-kW<sub>e</sub> systems designed for the near-Earth micrometeoroid environment and for a TIT of 1325°K (1925°F). These systems would be based on essentially existing technology (mostly superalloy with some well-characterized refractory hot section components) which results in relatively large masses and radiator areas. The system configuration with a three-panel radiator is shown previously in Figure 4 and fits easily within the Shuttle payload envelope.

The specific mass and radiator areas characteristic for 1000-kW<sub>e</sub> systems are shown in Figures 11 and 12. The TIT for these systems, 1650°K (2510°F), is commensurate with ceramic technology which should be available by 1995 (yielding a projected operational date of 2000). Specific mass and radiator area are the lowest of all the systems analyzed, illustrating most significantly the payoff of advanced technology. As a result of a number of on-going programs (four to six), AiResearch concludes that the ceramic technology will be available as outlined above. Indeed, these time frame projections may be conservative rather than optimistic.



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Table 2 shows the effect of environment, power level, and turbine inlet temperature on specific mass and radiator area. The table lists the specific radiator area of the system that has minimum specific mass. The table shows that both specific mass and radiator area decrease as power level and/or turbine inlet temperature increase. The requirement to operate in the Jovian environment has an adverse effect on system mass.

In Table 2, the values in the column labeled "Specific System Mass" were determined directly from the "shotgun" plots. As noted previously, the radiator is by far the most massive component in the power system. As described in Section 2.2.5, a modification to use "dual-diameter" heat pipes was defined late in the study. In this approach, a large diameter is used in the evaporator section to yield adequate heat transfer from the cycle working fluid and a smaller diameter is used for the condenser. This modification allowed the radiator to be redesigned for a much lower mass. Estimates were made of the reductions possible, and the resultant modifications to the specific mass are listed in the column labeled "Specific System Mass with Refined Radiator". The plots may still be used to determine the relative merits of alternative system design points.

The most important parameter in Table 2 is the specific mass of the 400-kW<sub>e</sub> system. For the Jovian environment, the specific mass is 21 kg/kW<sub>e</sub> which is within 5 percent of the design goal. With further refinement, this value could probably be made less massive than the goal. 400 kW<sub>e</sub> systems designed for the near-Earth environment are lighter than the design goal of 20 kg/kW<sub>e</sub>. The advanced 1000-kW<sub>e</sub> system for Jovian environment has the highest performance of all with a specific mass of 15 kg/kW<sub>e</sub>.



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**TABLE 2**  
**SUMMARY OF SELECTED BRAYTON SYSTEM DESIGNS**

Power Level $\text{kW}_e$	Turbine Inlet Temperature $^{\circ}\text{K}$	Environment	Specific System Mass $\text{kg/kW}_e$	Specific System Mass With Refined Radiator $\text{kg/kW}_e$	Specific Radiator Area $\text{m}^2/\text{kW}_e$
400	1500	Jovian	28	21	0.42
400	1500	Near-Earth	23	19	0.42
100	1325	Jovian	46	41	1.0
100	1325	Near-Earth	34	30	1.0
100	1500	Near-Earth	29	26	0.72
1000	1500	Jovian	26	20	0.38
1000	1650	Jovian	21	17	0.30
1000	1800	Jovian	18	15	0.25



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## 2.2 Task 2 - Primary Radiator Conceptual Design

In this task, primary waste heat radiator layout studies were undertaken. An analytical model of the radiator was defined and used in Task 1 to create the designs. Finally, the radiator design was checked against the Thermacore heat-pipe data. The geometry of the radiator selected for the 400-kW<sub>e</sub> system is discussed in Section 2.3.5 below.

### 2.2.1 Configuration Studies

The radiator configurations derived during the layout study are shown in Figure 13. There are four designs labeled (A) through (D).

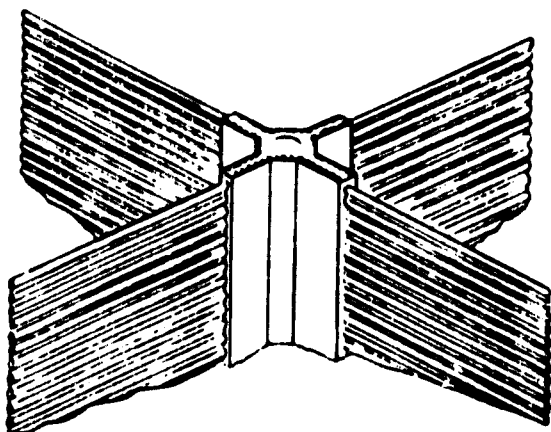
All of these configurations are similar in their design approach. Each has a gas-to-heat-pipe heat exchanger where the heat is transferred to the evaporator section of the heat pipe. Each has a condensing section and fin from which the waste heat is radiated. Each has armor protection for the condenser section. Some radiate from only one side of the panel.

Configuration (A) is based on a LASL design approach. It has four heat-pipe panels with a cruciform gas to heat pipe heat exchanger. Inside the heat exchanger are small diameter tubes which are joined to the evaporator. These tubes carry the working fluid and provide additional heat transfer surface.

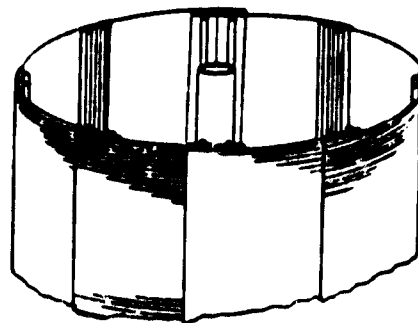
Configuration (B) has eight slightly curved panels which together make a right circular cylinder. Each panel has two gas-to-heat-pipe heat exchangers because these are two power conversion systems. Each gas heat exchanger is protected from meteoroid impact by the adjacent heat-pipe panels that overlap the heat exchangers.



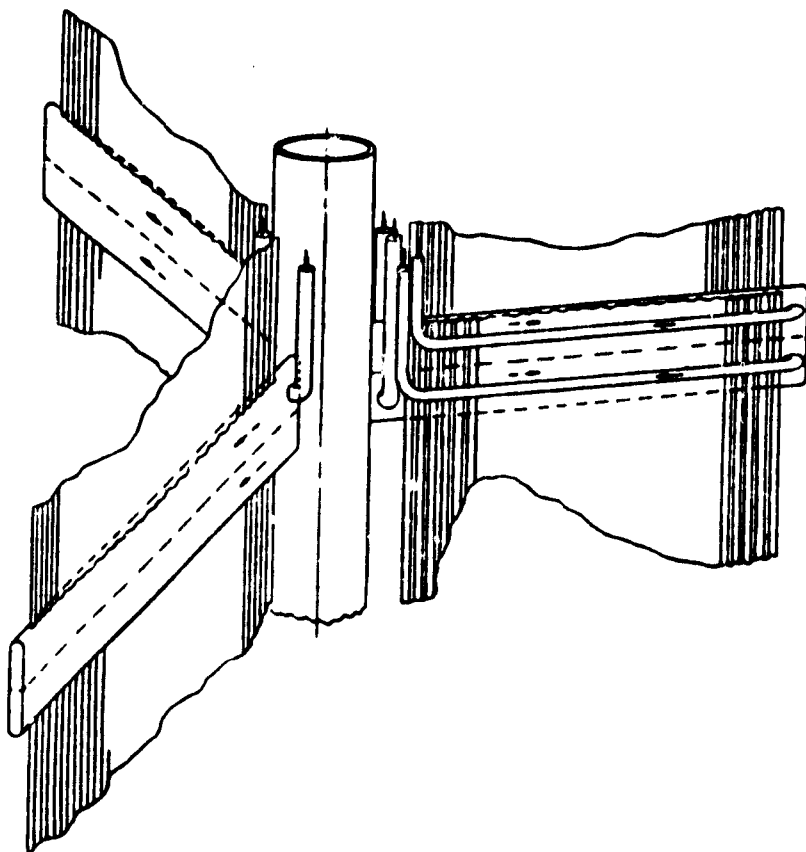
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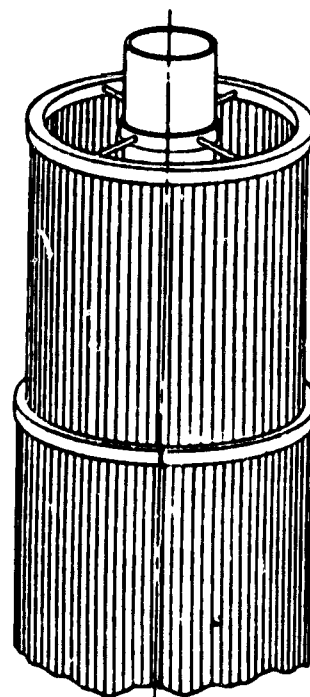
**A. FOUR PANEL (LASL) DESIGN**



**B. CYLINDRICAL DESIGN**



**C. THREE PANEL DESIGN**



**D. CAMPING-CUP DESIGN**

**Figure 13. Conceptual Primary Radiator Designs**



Configuration (C) is a three-panel design with axial heat pipes. There is a double heat exchanger located midway the axial length of the panel. This heat exchanger has heat pipes that exit from both the top and bottom.

The last radiator concept configuration (D), has a number of telescoping panels. The heat pipes are parallel to the axis of the radiator. The gas heat exchanger is located at the top of each panel. The major problem with this design is the flexible joints that must seal the working fluid loop.

For this study, configuration (B) was selected for the 400-kW<sub>e</sub> system. A combination of the features of (A) and (C) were used for the 100-kW<sub>e</sub> layout.

### 2.2.2 Meteoroid Protection

The armor thickness is based on the equation shown on Figure 14. This information was supplied by JPL and includes the effect of operation in the Jovian environment. The number of failures is based upon the binomial distribution. For this study, it was found advantageous to design for a high probability of non-puncture. Table 3 lists shows the effect of non-puncture probability on armor thickness.

TABLE 3

#### THE EFFECT OF NON-PUNCTURE PROBABILITY ON ARMOR THICKNESS

<u>Probability of Non-puncture of An Individual Tube</u>	<u>Percentage of Tubes Not Punctured*</u>	<u>Armor Thickness Ratio</u>
0.85	83.5	1.000
0.90	88.8	1.134
0.95	94.1	1.397
0.99	98.6	2.242

\*99 percent probability that no more than this percentage of tubes will be punctured. The specific design for the 400-kW<sub>e</sub> system was based on 95 percent non-puncture.





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- BASED ON PROTECTION FOR A SINGLE HEAT PIPE

$$t = C \left[ \frac{AT}{-ln(P)} \right]^{0.2902}$$

WHERE  $t$  IS ARMOR THICKNESS, cm  
 $C$  IS 0.00110 FOR LOCKALLOY  
 $A$  IS VULNERABLE AREA,  $cm^2$   
 $T$  IS MISSION TIME, HOURS  
 $P$  IS NO PENETRATION PROBABILITY

- NUMBER OF HEAT PIPE FAILURES BASED ON BINOMIAL DISTRIBUTION

$$P(i) = \sum_{m=0}^i \frac{p^m (1-p)^{n-m} n!}{m! (n-m)!}$$

$P(i)$  IS PROBABILITY OF  $i$  OR FEWER PUNCTURES ( $i \leq n$ )  
 $n$  IS TOTAL NUMBER OF HEAT PIPES  
 $m$  IS NUMBER OF FAILED HEAT PIPES

Figure 14. Meteoroid Protection Criteria



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It can be seen that increasing the non-puncture probability from 0.85 to 0.95 increases the armor thickness (and mass) by 40 percent. This is a worthwhile tradeoff when the major mass is the heat exchanger and heat pipes. If a low probability is selected, more heat pipes and heat exchanger mass must be added to compensate for the failed heat pipes. Excess heat transfer area on both the gas convection surface and the radiating surface will ensure that this radiator will function as designed when 6 percent of the tubes have failed.

### 2.2.3 Analytical Design

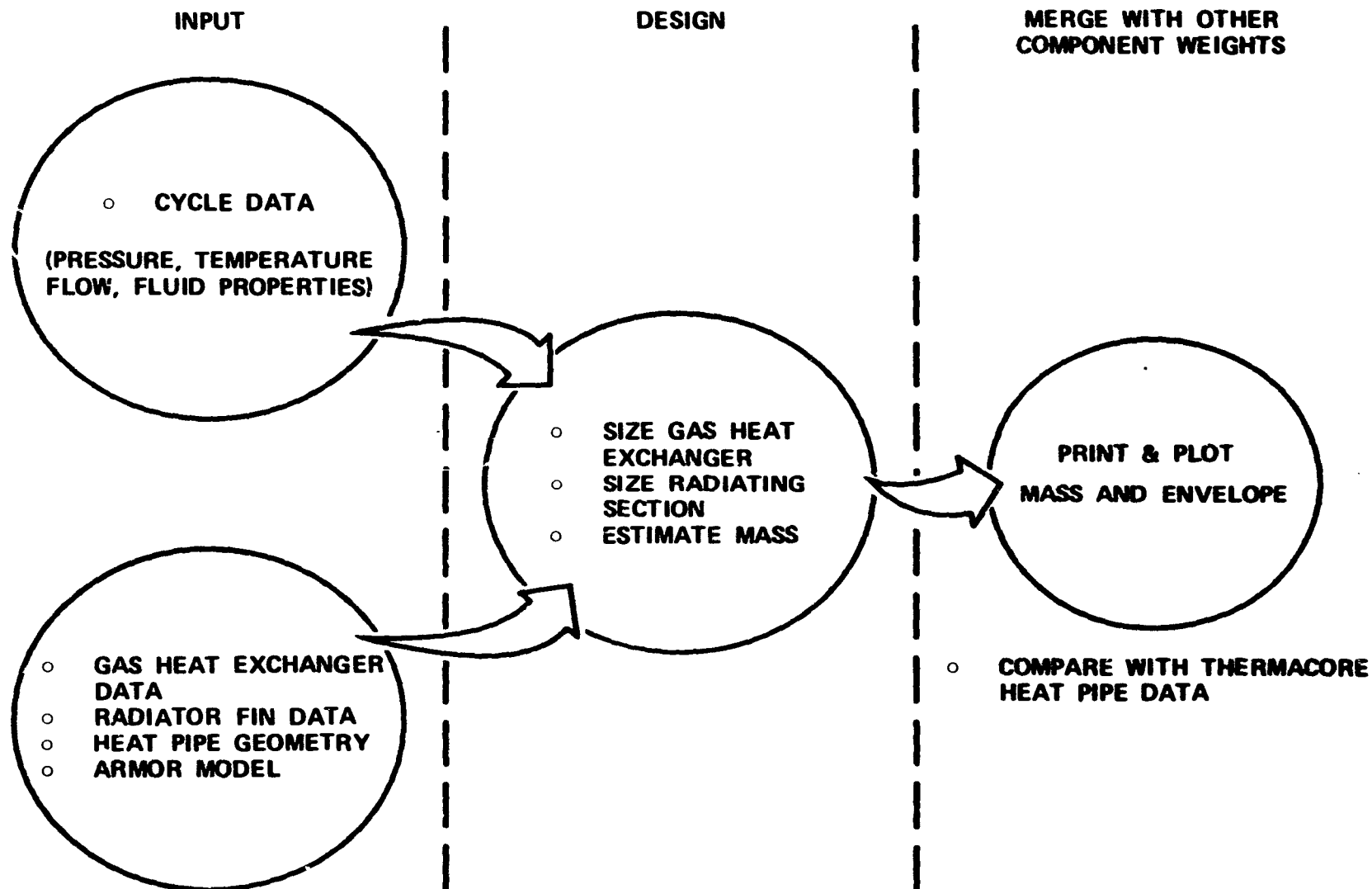
Analysis of the radiator utilizes a computer program. This program has been integrated into the cycle design program so that radiators can be designed at the same time the thermodynamic cycle is derived. Figure 15 summarizes the design method. Input from the cycle design program constitutes the major variables, such as:

- o Heat rejection rate
- o Flow
- o Temperature
- o Fluid properties
- o Pressure drop

Other input, which applies to the particular configuration to be studied, is read by the radiator program. This input includes:

- o Heat pipe diameter and wall thickness
- o Condenser length
- o Gas heat exchanger width
- o Heat transfer data
- o Armor model

With this input, the gas heat exchanger size and radiating area are calculated. The frontal area of the heat exchanger (and therefore



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Figure 15. Radiator Design Method



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the evaporator length) is a function of the pressure drop. The heat transfer conductance has a strong effect on radiator size. The computer program calculates the length of the heat exchanger based on the conductance and radiating temperature. Evaporator length is changed as necessary to satisfy pressure drop. The program iterates until both heat transfer and pressure drop requirements are satisfied simultaneously.

When the calculation has converged, the mass of the radiator is calculated. This mass includes heat pipes, fin, armor, two gas-to-heat-pipe heat exchangers, and associated heat exchanger components (wrapup, headers, duct extension, and flanges).

The last step is to compare the computer heat pipe results with data on specific designs supplied by Thermacore. If the result is favorable, the conceptual design is accepted. If not, iteration is required.

#### 2.2.4 Heat-Pipe Data

Appendix A includes the results of the Thermacore radiator heat pipe study (7). The important conclusions are:

- o Rubidium is the preferred working fluid for 1 in. OD heat pipes when the temperature is above 650°K (710°F).
- o Mercury is acceptable for temperatures as low as 550°K (530°F) if the heat-pipe diameter is less than 1 in. In fact, the power transferring capability of the heat pipe increased as the diameter decreased.
- o Dowtherm A is the preferred fluid below 550°K (530°F) [minimum radiating surface temperature is 492°K (426°F)].
- o Other advanced designs might offer increased performance and lower mass.



### 2.2.5 Advanced Heat-Pipe Concepts

A comparison of the performance characteristics of the evaporator and condenser sections of constant diameter heat pipes led to the conclusion that this type of heat pipe was not the optimum design. Thermacore agreed that it was possible to design and build heat pipes in which the evaporator diameter is greater than the condenser diameter. This results in a heat-pipe radiator in which the evaporation area is sufficient for convective heat transfer from the gas working fluid, and the condenser section is operated somewhat closer to the maximum heat transfer capability. The smaller heat-pipe condenser section minimizes vulnerable area and heat-pipe mass; consequently, armor mass is also minimized. This design concept reduced the radiator mass of the 400-kW<sub>e</sub> reference system by 45 percent. Use of alternative advanced heat-pipe geometries may provide further substantial mass savings.

### 2.3 Task 3, Reference System Configuration and Component Conceptual Design

The overall reference system design concept is described in Section 2.1.3 above. The 400-kW<sub>e</sub> spacecraft design that will fit in the Space Shuttle orbiter payload bay is shown in Figure 3 with major elements identified. The mass summary discloses the radiator to be, by far, the major mass element of the 8270-kg total mass. Further refinement of this heat-pipe radiator design and of the other components will permit appreciable reduction in the specific mass of 20.7 kg/kW<sub>e</sub>.

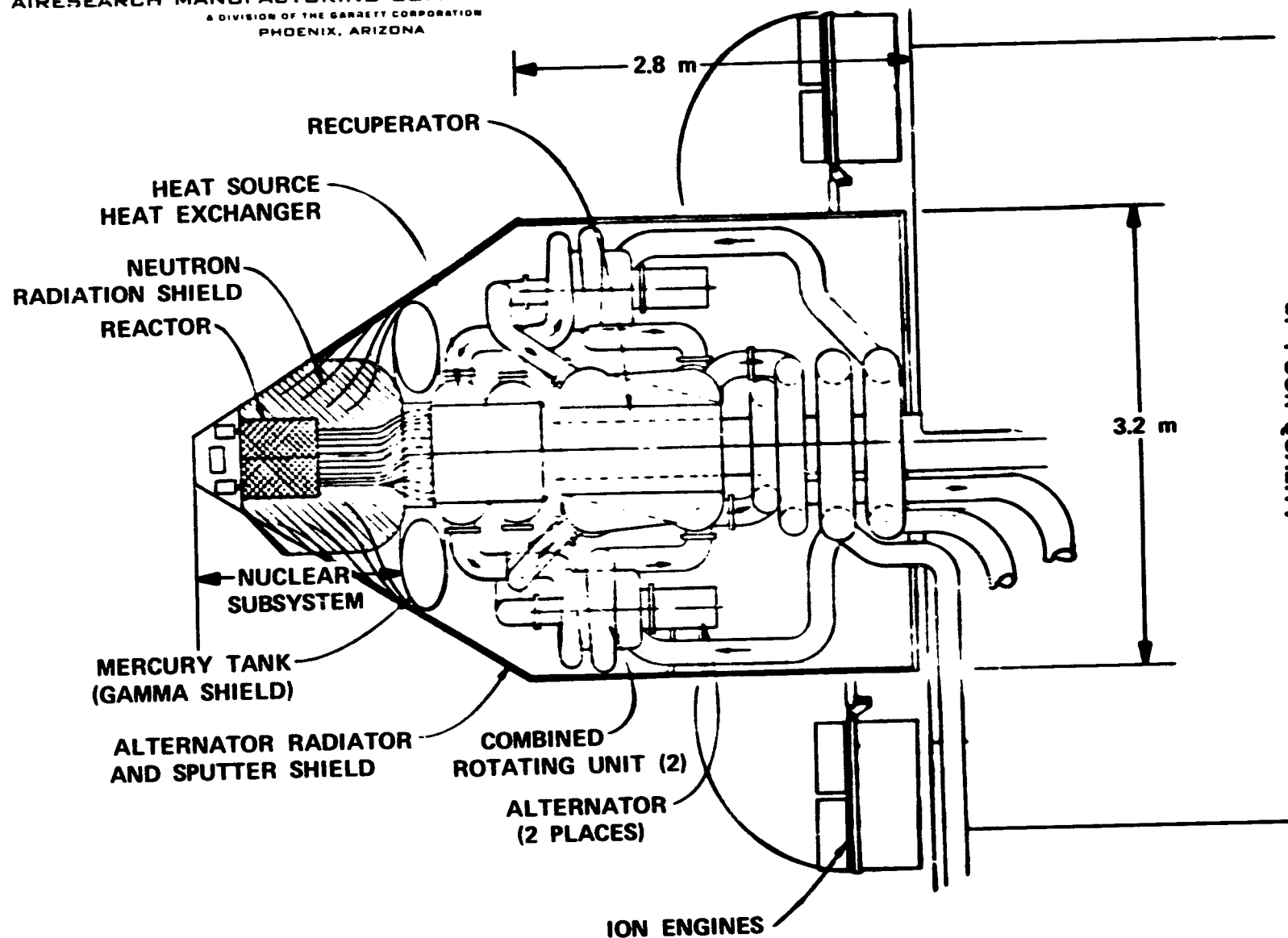
#### 2.3.1 Reference System Configuration

The configuration of the 400-kW<sub>e</sub> Brayton power system is shown in Figure 16\* with the nuclear subsystem farthest aft, away from the

\*Approximate dimensions for the power conversion components may be scaled from the figure.



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Figure 16. Configuration of 400-kW<sub>e</sub> Brayton Power System  
For a Nuclear Electric Spacecraft.

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spacecraft, and the compactly clustered components of the two power conversion systems next to the panel-mounted ion engines. The cylindrical radiator encloses the spacecraft during launch and, thus, is between the spacecraft and the power system in the deployed configuration.

A schematic of the dual Brayton power systems used to eliminate the single-point failure mode in the nuclear electric spacecraft is given in Figure 17. Both completely independent systems, each of which is capable of producing  $400 \text{ kW}_e$ , are operated at half power under normal conditions. In the unlikely event that one of the systems becomes inoperable, the failed system is turned off and the pressure level doubled in the remaining system to restore full power output. Other means of assuring the required reliability for long durations are conceivable but the above method is preferred, at least until quantitative reliability and mass trade-off studies are accomplished.

Brayton cycle state points for the  $400\text{-kW}_e$  reference system are given in Figure 18. The reactor with an outlet temperature of  $1600^\circ\text{K}$  provides the thermal energy to the heat source heat exchanger that provides the temperature rise from  $1114$  to  $1500^\circ\text{K}$ . The primary radiator provides the compressor inlet temperature of  $500^\circ\text{K}$ . The mass flow is  $7.5 \text{ kg/sec}$ .

### 2.3.2 Nuclear Subsystem

The nuclear subsystem configuration is delineated in Figure 19. The two major components are the reactor and its lithium hydride neutron shield. Controls, insulation, shield cooling heat pipes and radiator, and the mercury propellant tank, which serves as a gamma shield, are also shown in the figure.



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NOTE: DUAL FULL POWER CONVERSION  
SYSTEMS ARE COMPLETELY INDEPENDENT

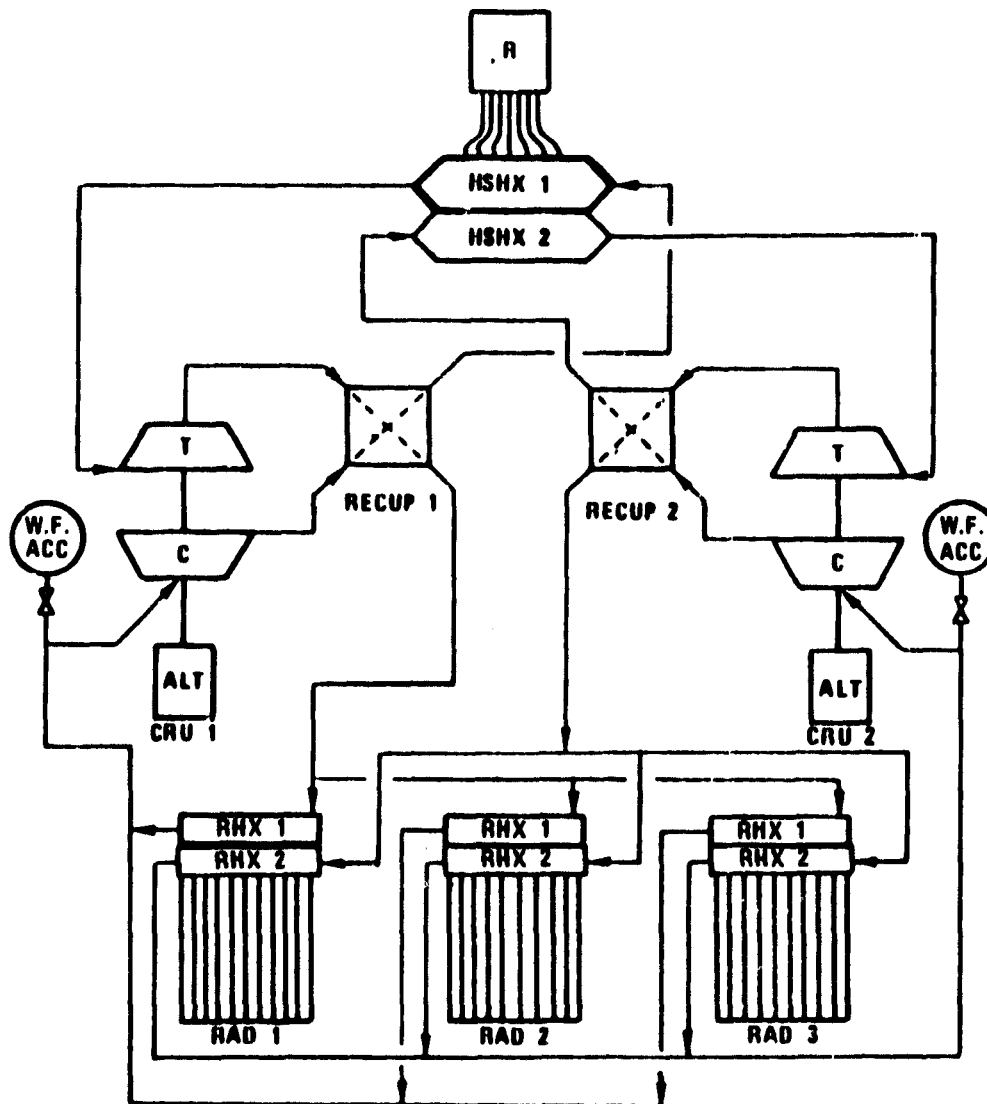


Figure 17. Nuclear Electric Spacecraft Dual Brayton Power Systems Schematic





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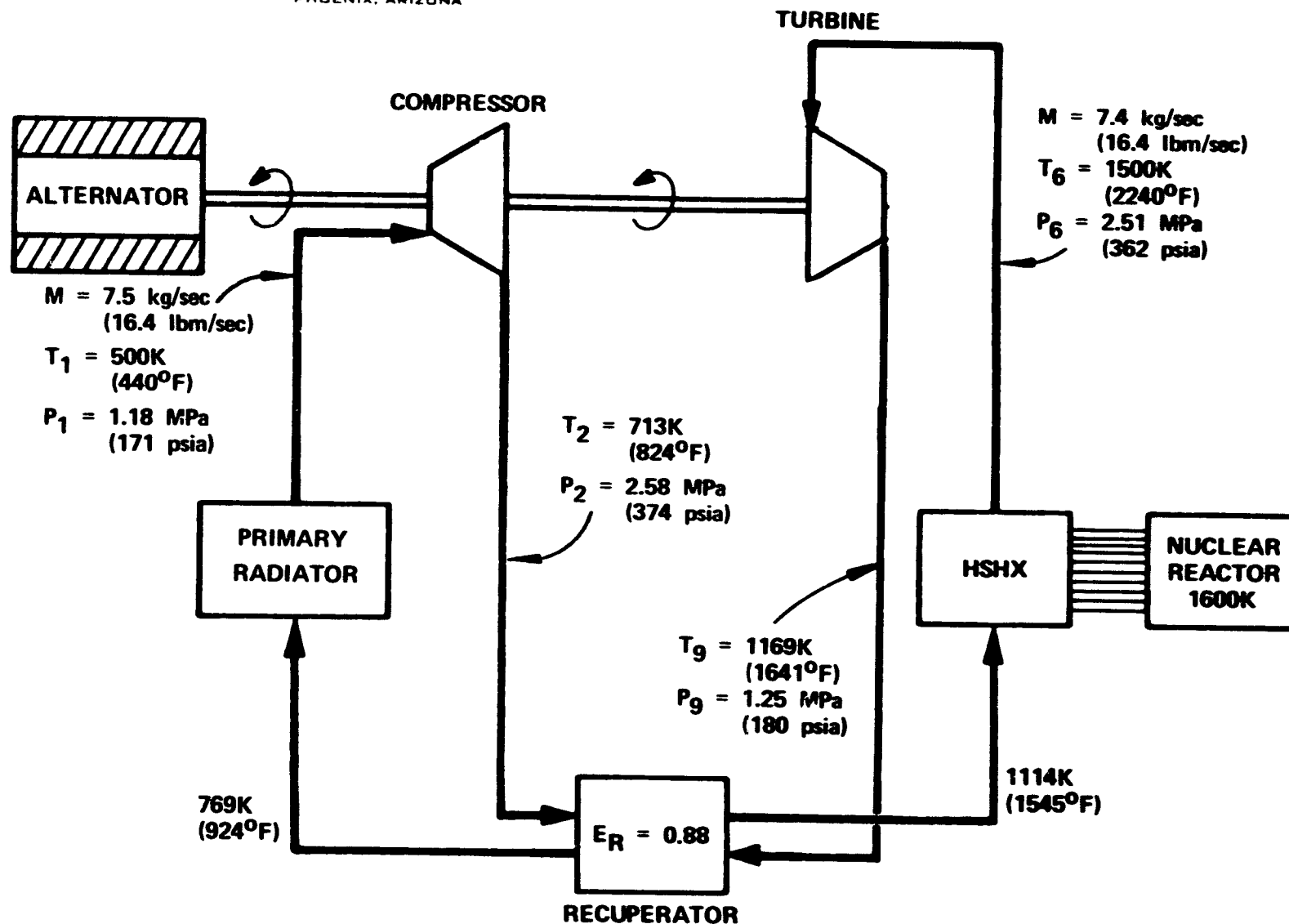


Figure 18. 400 kW<sub>e</sub> Reference Power System Brayton Cycle State Points

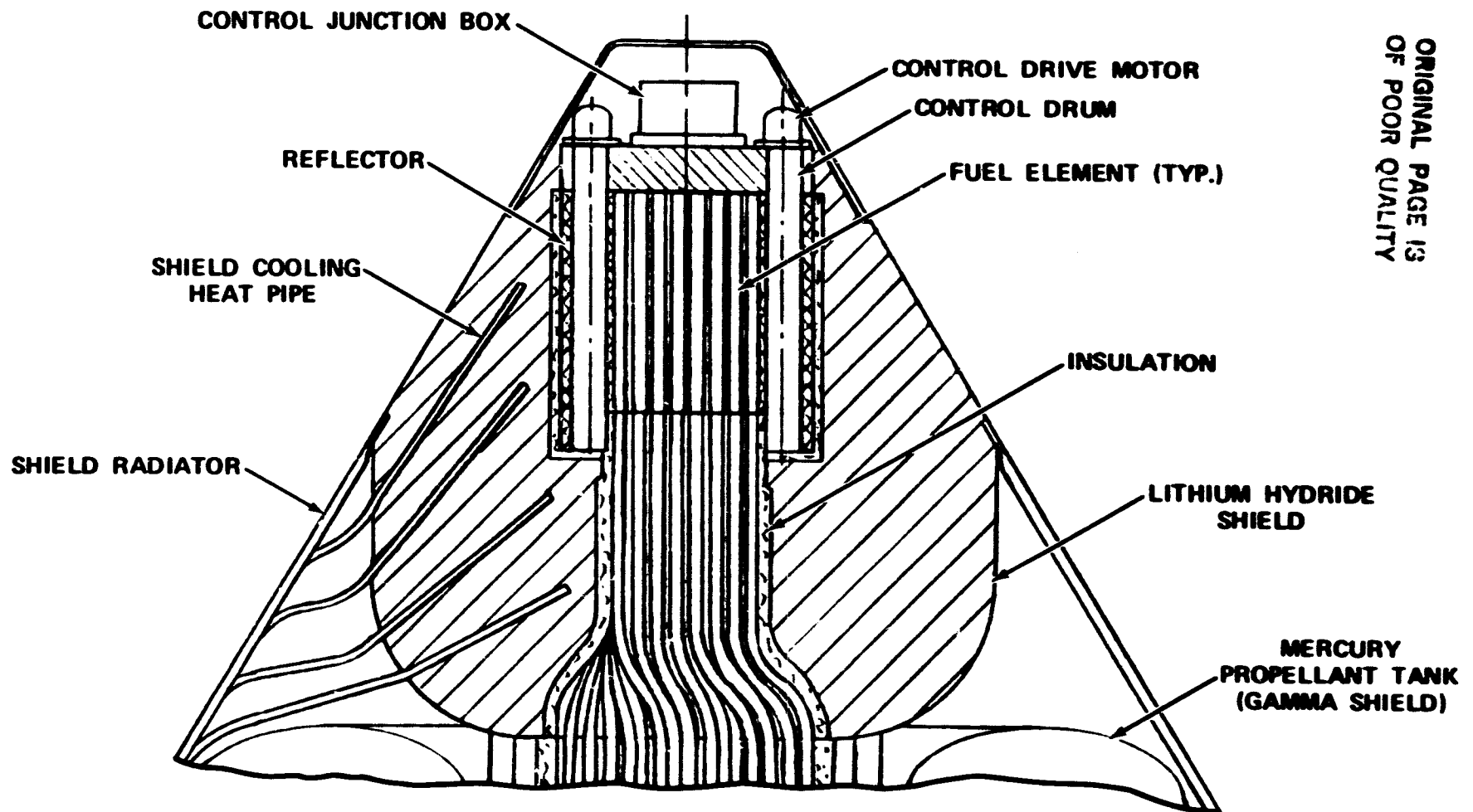
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Figure 19. Space Nuclear Subsystem Reference Configuration



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The relation of heat-pipe cooled reactor characteristics to the characteristics of the nuclear subsystem are diagrammed in Figure 20. Preliminary determinations have been made of reactor and subsystem specific mass; the other characteristics remain for future analysis.

### Reactors

LASL has provided parametric data on heat-pipe cooled reactors with uranium oxide ( $\text{UO}_2$ ) and uranium carbide (UC) fuels. These data (8,9) are included in Appendix B. Data were also provided on gas cooled reactors (10) but they have not been used in this study. Characteristics of the nominal 1650-kW<sub>t</sub> reactor selected for the 400-kW<sub>e</sub> reference power system are shown in Table 4. The hexagonal fuel elements are made from a 60%  $\text{UO}_2$ -40% molybdenum mixture and have a central heat pipe for removing the thermal power. This reactor is a 0.6 m "square" cylinder with a total mass of 875 kg. The reactor specific mass is 0.53 kg/kW<sub>t</sub>. The design concept for this LASL reference reactor is shown in Figure 21.

Another reference reactor with 400-kW<sub>t</sub> nominal thermal power was identified for the preliminary conceptual design of the 100-kW<sub>e</sub> power system with the characteristics listed in Table 5. This reactor has Uranium carbide-Zirconium carbide fuel elements. It has a 0.24-m diameter and a 0.24-m length, and a mass of 346 kg so that the specific mass is 0.87 kg/kW<sub>t</sub>. The configuration of the 400-kW<sub>t</sub> reactor can be seen in Figure 22 to be similar to the 1650-kW<sub>t</sub> reactor.

LASL has recently published data on a new layered core heat-pipe cooled reactor design concept (11). The 1200-kW<sub>t</sub> thermal power version is shown in Figure 23. Data on these new reactors have been made available too recently to be incorporated in this study although the layered core offers many improved characteristics. The reactor mass at 1200 kW<sub>t</sub> is currently given as 470 kg for a specific mass of 0.39 kg/kW<sub>t</sub> which should result in considerably improved system parameters.

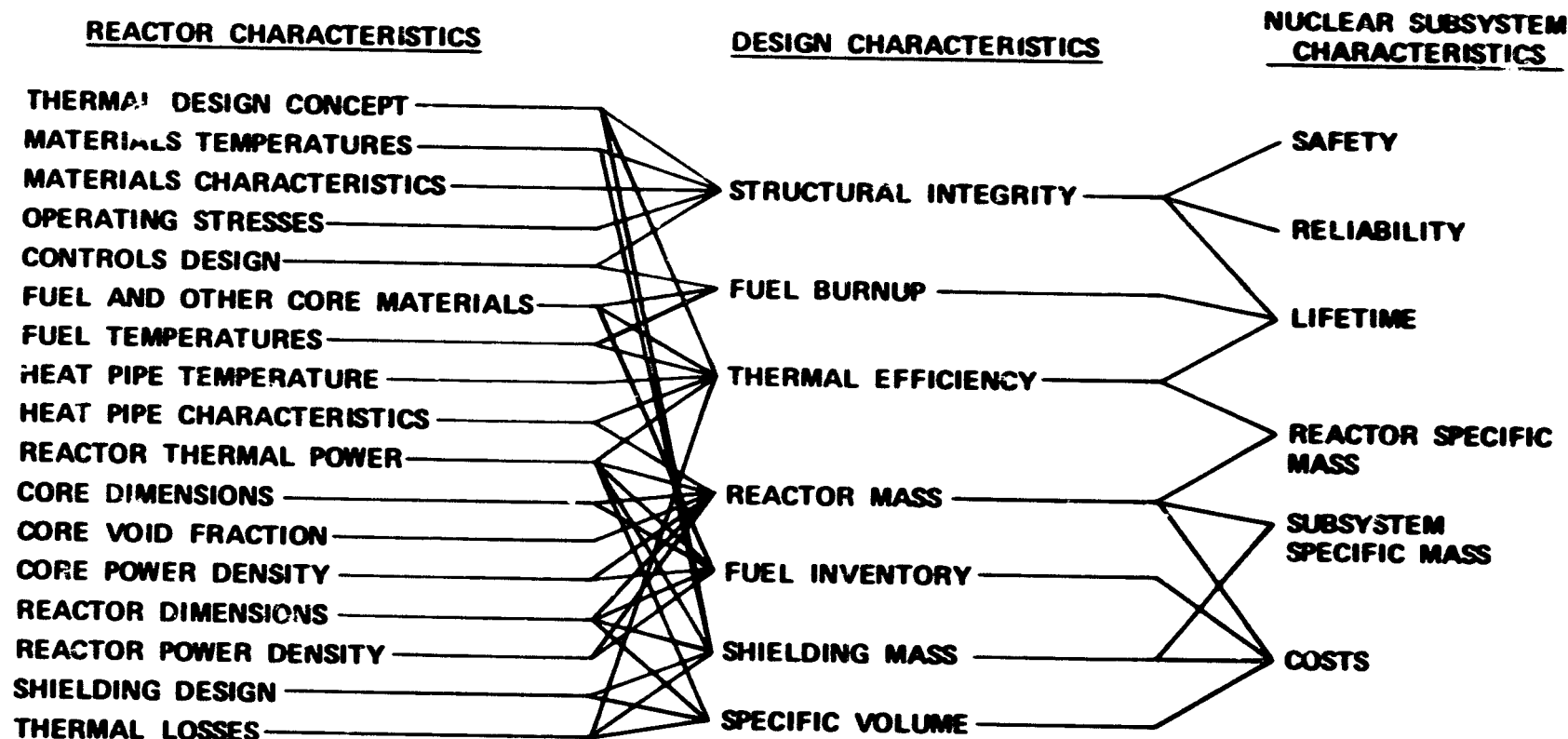
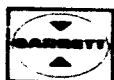


Figure 20. Relation of Heat Pipe Cooled Reactor Characteristics to Nuclear Subsystem Characteristics



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TABLE 4  
LASL REFERENCE REACTOR CHARACTERISTICS - 1650 kW<sub>t</sub> (nom.)

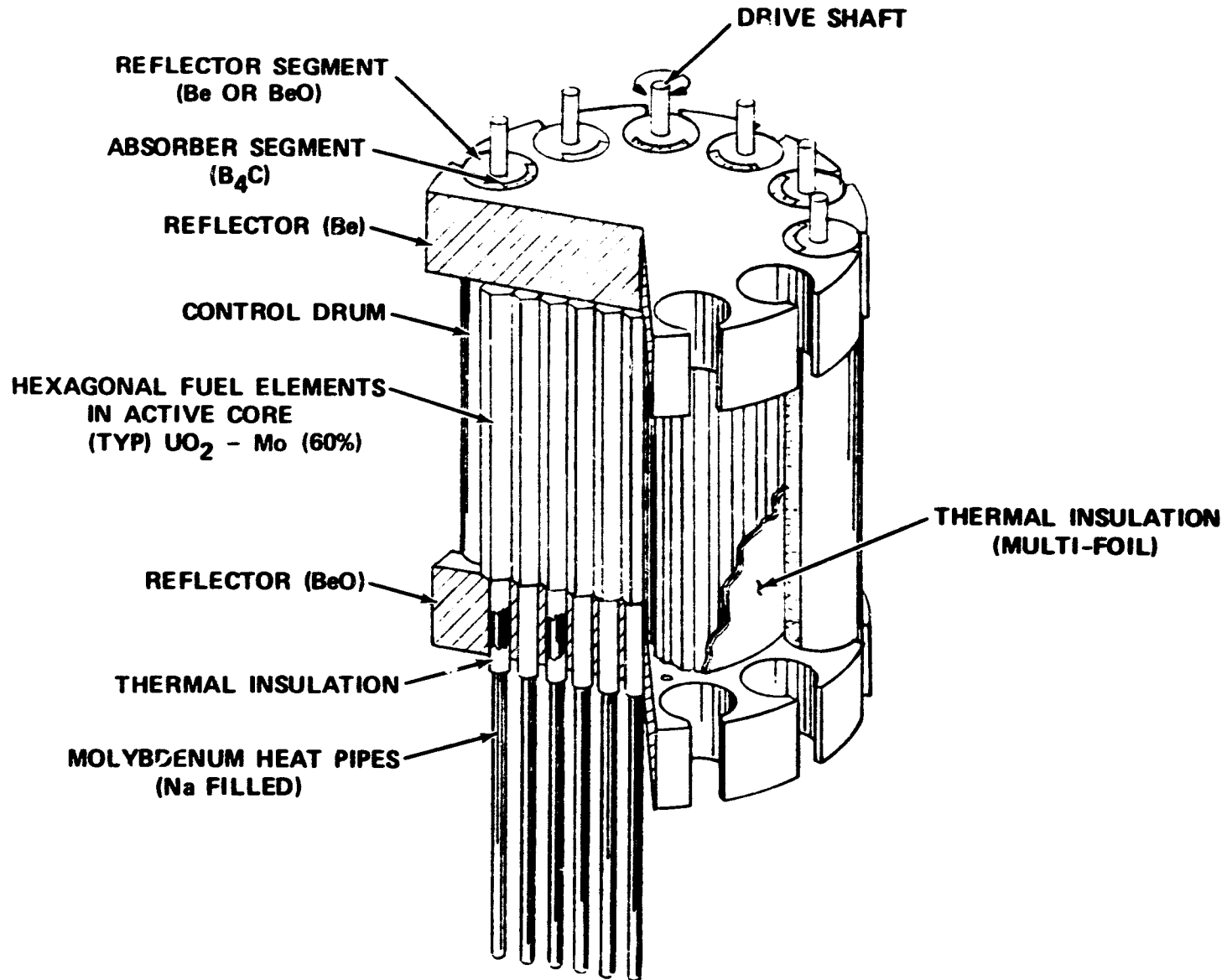
REACTOR TYPE	FAST SPECTRUM, HEAT PIPE COOLED
THERMAL POWER	1650 kW <sub>t</sub>
LIFETIME	87.6 X 10 <sup>3</sup> h
FUEL TYPE	UO <sub>2</sub> - Mo (60%)
FUEL ELEMENT CLAD	Mo
HEAT PIPE WALL/WORKING FLUID	Mo/Li
NO. OF FUEL ELEMENTS (AND HEAT PIPES)	210
FUEL ELEMENT WIDTH (ACROSS HEX. FLATS)	0.0246 m
FUEL ELEMENT LENGTH	0.2756 m
MAXIMUM FUEL TEMPERATURE	1696 K
AVERAGE POWER IN FUEL SPACE	53 MW <sub>t</sub> /m <sup>3</sup>
FISSION DENSITY	4.968 X 10 <sup>20</sup> FISSIONS/cm <sup>3</sup>
FUEL SWELLING	1.11 VOLUME %
FUEL ( <sup>235</sup> U) BURNUP	4.6 ATOM %
CORE DIAMETER	0.3756 m
CORE LENGTH	0.3756 m
CORE VOLUME	0.042 m <sup>3</sup>
CORE VOID FRACTION	0.328
CORE POWER DENSITY	39.3 MW <sub>t</sub> /m <sup>3</sup>
REFLECTOR MATERIAL	Be, BeO
STRUCTURAL MATERIAL	Mo
REACTOR DIAMETER	0.6056 m
REACTOR HEIGHT	0.5856 m
REACTOR VOLUME	0.169 m <sup>3</sup>
REACTOR POWER DENSITY	9.76 MW <sub>t</sub> /m <sup>3</sup>
REACTOR OUTLET (HEAT PIPE) TEMPERATURE	1600 K
REACTOR MASS SUMMARY, kg	
FUEL ( <sup>235</sup> U MASS = 150.6 kg)	275
REFLECTOR	322
HEAT PIPES (112.24 kg/m)	166
CONTROL SYSTEM	33
SUPPORT STRUCTURE	57
TOTAL MASS	875
REACTOR SPECIFIC MASS	0.530 kg/kW <sub>t</sub>

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Figure 21. LASI Reference Reactor Design Concept - 1650 kW<sub>t</sub> (nom.)

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TABLE 5  
LASL REFERENCE REACTOR CHARACTERISTICS - 400 kW<sub>t</sub> (nom.)

REACTOR TYPE	FAST SPECTRUM, HEAT PIPE COOLED
THERMAL POWER	400 kW <sub>t</sub>
LIFETIME	87.6 X 10 <sup>3</sup> h
FUEL TYPE	UC - ZrC
FUEL ELEMENT CLAD	Mo
HEAT PIPE WALL/WORKING FLUID	Mo/Li
NO. OF FUEL ELEMENTS (AND HEAT PIPES)	84
FUEL ELEMENT WIDTH (ACROSS HEX FLATS)	0.0246 m
FUEL ELEMENT LENGTH	0.2381 m
MAXIMUM FUEL TEMPERATURE	1554.2 K
AVERAGE POWER IN FUEL SPACE	47.83 MW <sub>t</sub> /m <sup>3</sup>
FISSION DENSITY	4.474 X 10 <sup>20</sup> FISSIONS/cm <sup>3</sup>
FUEL SWELLING	6.44 VOLUME %
FUEL BURNUP	2.15 ATOM %
CORE DIAMETER	0.2381 m
CORE LENGTH	0.2381 m
CORE VOLUME	0.0106 m
CORE VOID FRACTION	0.3295
CORE POWER DENSITY	37.74 MW <sub>t</sub> /m <sup>3</sup>
REFLECTOR MATERIAL	Be, BeO
STRUCTURAL MATERIAL	Mo
REACTOR DIAMETER	0.4681 m
REACTOR LENGTH	0.4481 m
REACTOR VOLUME	0.0771 m
REACTOR POWER DENSITY	5.188 MW <sub>t</sub> /m <sup>3</sup>
REACTOR OUTLET (HEAT PIPE) TEMPERATURE	1425 K
REACTOR MASS SUMMARY, kg	
FUEL ( <sup>235</sup> U MASS = 78.2)	92.0
REFLECTOR	162.1
HEAT PIPES	36.5
CONTROLS	33.0
SUPPORT STRUCTURE	22.7
TOTAL MASS	346.3
REACTOR SPECIFIC MASS	0.686 kg/kW <sub>t</sub>

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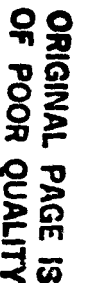


Figure 22. LASL Reference Reactor Design Concept - 400 kW<sub>t</sub> (nom.)





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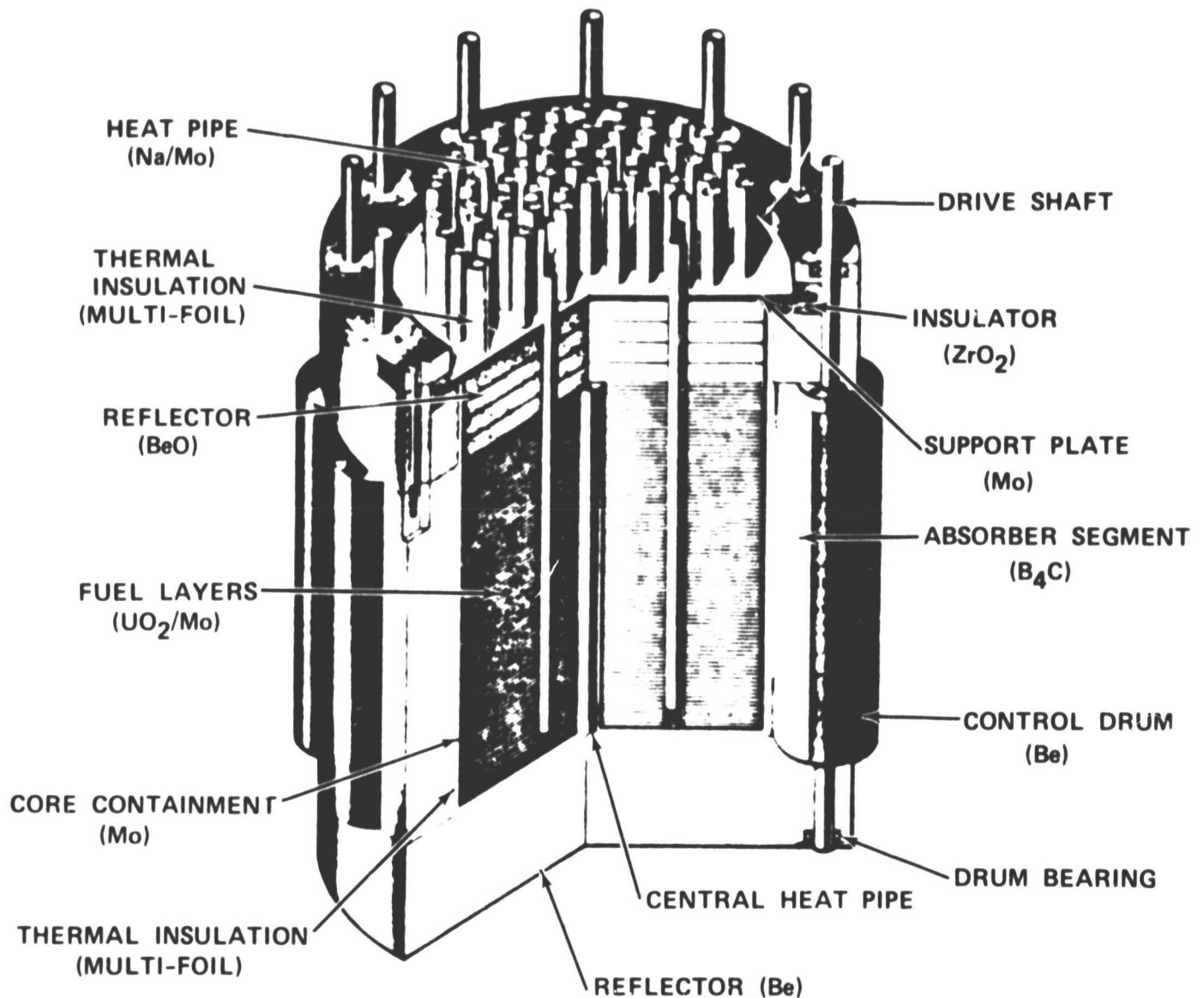


Figure 23. LASL Layered Core Heat Pipe Cooled Space Power Reactor Design Concept - 1200 kW<sub>t</sub>.



### Radiation Shields

The nuclear radiation shielding has not been given detailed attention since the requirement is similar to the design used for the thermionic systems. Tailoring of the lithium hydride neutron shield to reduce its mass is shown in Figure 19. The propellant tank is located so that it can be used as a gamma radiation shield and is sized to hold an additional quantity of mercury for this purpose.

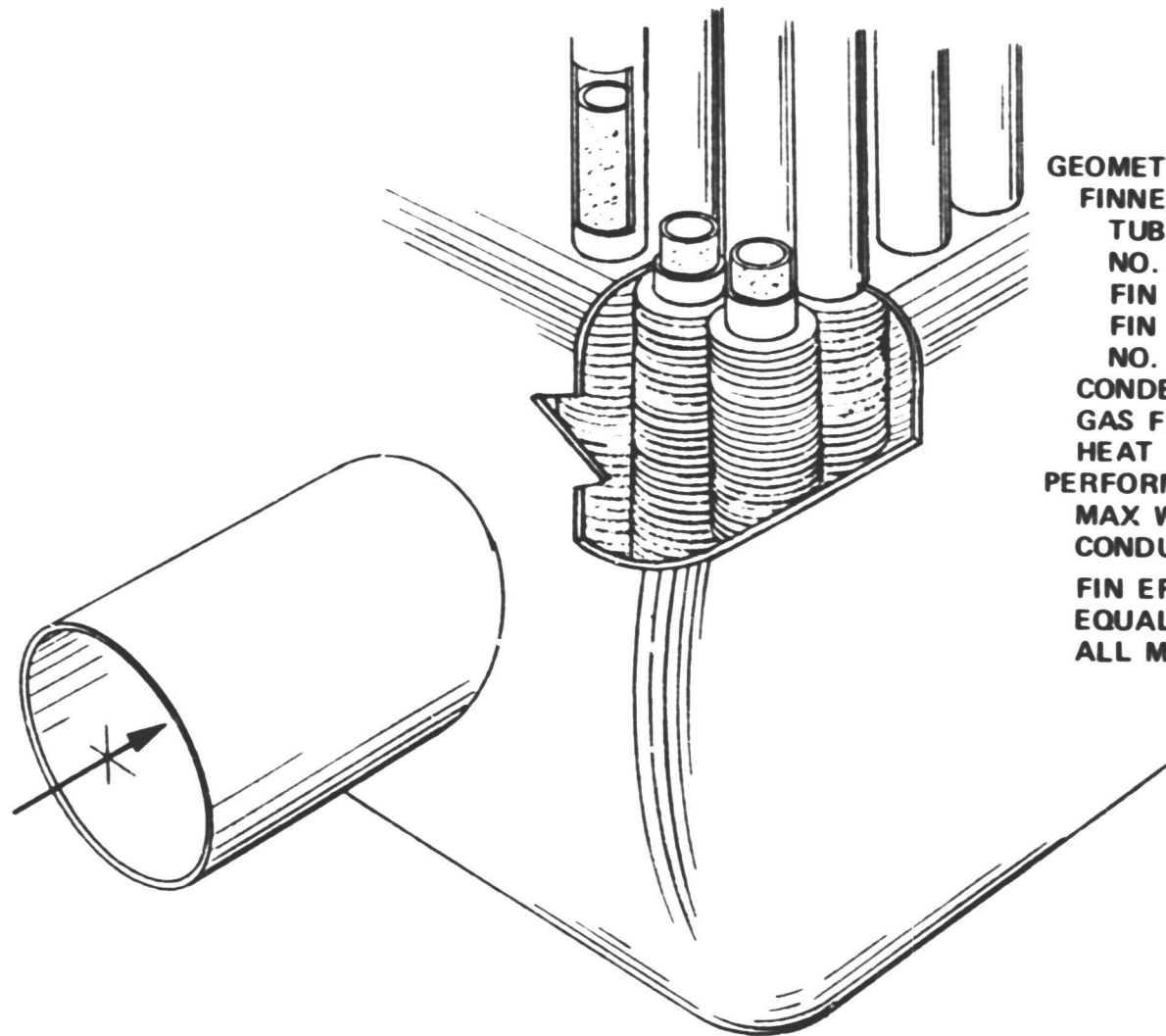
#### 2.3.3 Heat Source Heat Exchanger

The heat source heat exchanger (HSHX) is the highest temperature component of the closed cycle system(s). This component looks like a typical tube fin heat exchanger and is illustrated in Figure 24. The geometry and performance are also summarized on this chart. The operation differs from a normal heat exchange because this design has equal heat flux from each tube. This occurs because each tube is actually the condensing end of a heat pipe. The heat pipe receives its heat in the nuclear reactor and gives it up in the heat exchanger. The heat is transferred to the working fluid by convection from the heat pipes and fins. In the reference system, the HSHX receives working fluid from the recuperator at 1114°K and provides it to the turbine at 1500°K.

There is a heat source heat exchanger for each power system. Because of the nature of heat pipes, when each evaporator section is operated at half power (the nominal operation), the maximum metal temperature is reduced from the 1583°K (the value when only one section is in use). This heat exchanger is all-molybdenum construction. This material is also used in the heat pipes.



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### CHARACTERISTICS

#### GEOMETRY:

##### FINNED TUBE

TUBE OD = 2.5 cm (1 in.)

NO. FIN/cm = 10 (25 fins/inch)

FIN LENGTH = 0.5 cm (0.2 in.)

FIN THICKNESS = 0.25 mm (0.010 in.)

NO. FINNED TUBES = 162

CONDENSING LENGTH = 41.2 cm (16.2 in.)

GAS FLOW LENGTH = 67.8 cm (26.7 in.)

HEAT EXCHANGER WIDTH = 33.0 cm (13 in.)

#### PERFORMANCE

MAX WALL TEMPERATURE = 1583°K (2390°F)

CONDUCTANCE = 21 kW/°K (11Btu/sec-°F)

FIN EFFECTIVENESS = 67 PERCENT

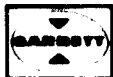
EQUAL HEAT FLUX PER TUBE

ALL MOLYBDENUM CONSTRUCTION

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Figure 24. Heat Source Heat Exchanger Concept and Characteristics



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#### 2.3.4 Combined Rotating Unit

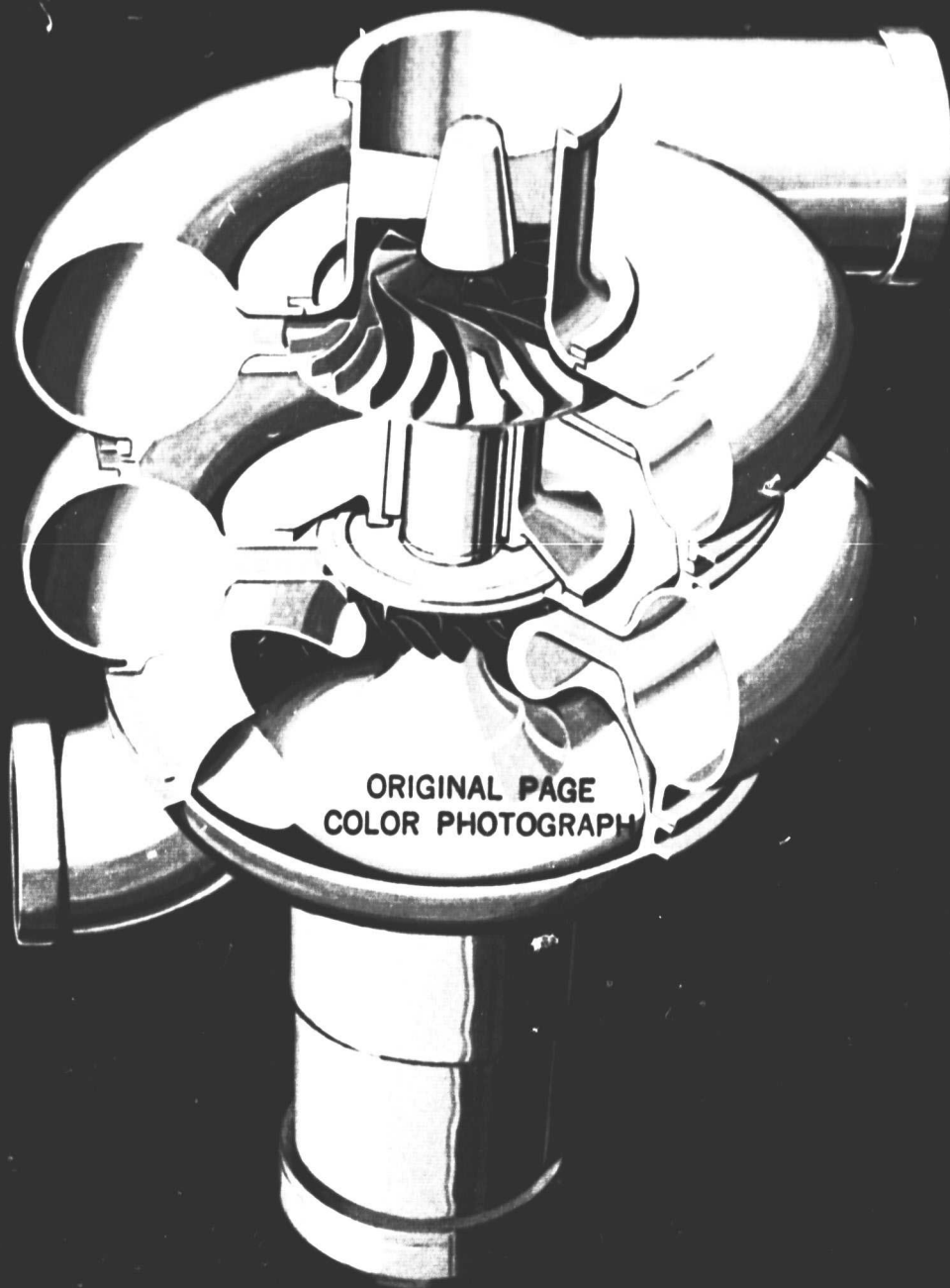
The combined rotating unit (CRU) is a highly efficient, single shaft, closed Brayton cycle design that has resulted from many years of development and test in industry and government, especially under NASA sponsorship. Figure 25 is a rendering of the engine. The three primary components are the compressor, turbine, and alternator.

The compressor is a state of the art design. It is radial outflow type with backward curved blades for maximum efficiency. The turbine is the radial inflow type with straight radial blades. It is also state of the art aerodynamic design.

Both the compressor and turbine are relatively small wheels. Moderate specific speed and relatively high Reynolds number result in high component efficiency. The actual design parameters are shown in Table 6.

Prior experience indicates that the CRU bearings are one of the most critical components for a long life space power system. The alternator is mounted on one pair of foil journal bearings. The compressor and turbine are mounted on a larger pair of similar bearings. Two sets of foil thrust bearings absorb the aerodynamic thrust of the compressor and turbine. Foil-type gas-lubricated bearings are designed such that all excursions are absorbed by a film of gas, which together with the foil, has a definable spring constant. With proper design, the rotor does not contact any bearing surface after it achieves a small fraction of the normal rotational speed during start. Because nothing rubs, there is no wear-out mode.

The alternator has a high performance samarium cobalt rotor. This rotor is smaller and lighter than a Rice alternator. Because it is smaller, the windage loss is much smaller. The alternator has the following performance:



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**TABLE 6****TURBINE AND COMPRESSOR WHEEL CHARACTERISTICS**

	<b>Turbine</b>	<b>Compressor</b>
<b>Pressure Ratio</b>	2.01	2.18
<b>Tip Speed</b>	440 m/sec (1440 ft/sec)	400 m/sec (1300 ft/sec)
<b>Mean Specific Speed</b>	62.0 rpm-ft <sup>3/4</sup> /sec <sup>1/2</sup>	58.5 rpm-ft <sup>3/4</sup> /sec <sup>1/2</sup>
<b>Tip Diameter</b>	23.4 cm (9.2 in.)	21 cm (9.3 in.)
<b>Reynolds Number</b>	380,000	24 x 10 <sup>6</sup>
<b>Mass</b>	11.4 kg (25 lb) Refractory or Advanced Superalloy	2.7 kg (6 lb) Titanium
<b>Efficiency</b>	91 Percent	86 Percent
<b>Comments</b>	<ul style="list-style-type: none"> <li>o State of the art aero dynamic design</li> <li>o Relatively small wheel with low tip speed</li> </ul>	<ul style="list-style-type: none"> <li>o High efficiency because of low pressure ratio, moderate specific speed and Reynolds number</li> <li>o Low risk aerodynamic design</li> </ul>

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Efficiency	96% without windage
Output	408 kW <sub>e</sub>
Frequency	3000 Hz
Voltage	500 Line-to-Neutral

The alternator is mounted adjacent to the compressor rather than the turbine so that it has a lower temperature environment. The alternator is cooled by heat pipes. The heat is dumped by the alternator heat pipe radiator.

### 2.3.5 Recuperator

The recuperator is the component which enables the high efficiency of closed Brayton cycle engines. It exchanges the heat from the turbine discharge gas to the colder gas leaving the compressor. Extensive recuperation leads to relatively low-pressure ratios in the turbomachinery. Low-pressure ratio yields simple, highly efficient compressors and turbines. The mass of the turbomachinery is also reduced. All of these beneficial results require increased recuperator mass. Shotgun plots such as Figures 5 through 12 illustrate this effect, revealing the best combination of components to be selected for each application.

The recuperator is shown in Figure 26. It is a pure counter flow design to minimize mass. Each flow is split and moves through alternate finned passages. The heat from the turbine discharge gas is exchanged by convection to the fin and wall and thence to the other gas stream. Each stream is completely sealed from the other.

The selected recuperator uses a conventional fin design of 8 fins/cm (20 fins/in.). The heat transfer data were derived from AiResearch development testing. Similar finning is used in present production units. The computer code uses this data together with the



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### CHARACTERISTICS

EFFECTIVENESS	- 88%
WELL WITHIN STATE-OF-THE-ART	
TOTAL PRESSURE DROP ( $\Delta P/P$ )	- 1.6%
OVERALL LENGTH	- 48 cm (19 INCHES)
HEIGHT	- 79 cm (31 INCHES)
WIDTH	- 41 cm (16 INCHES)
MASS	- 385 kg (805 lb <sub>m</sub> )
HEAT TRANSFER RATE	- 1552 kW <sub>t</sub> (1471 BTU/SEC)
MATERIAL	- HOT END MUST BE REFRACTORY (Cb)
	- COLD END COULD BE SUPERALLOY (HAST-X)
OTHER ASSEMBLY OPTIONS	- REFRACTORY HEAT PIPE HOT END, HAST-X
	USED AT TEMPERATURES BELOW 1033°K (1400°F)

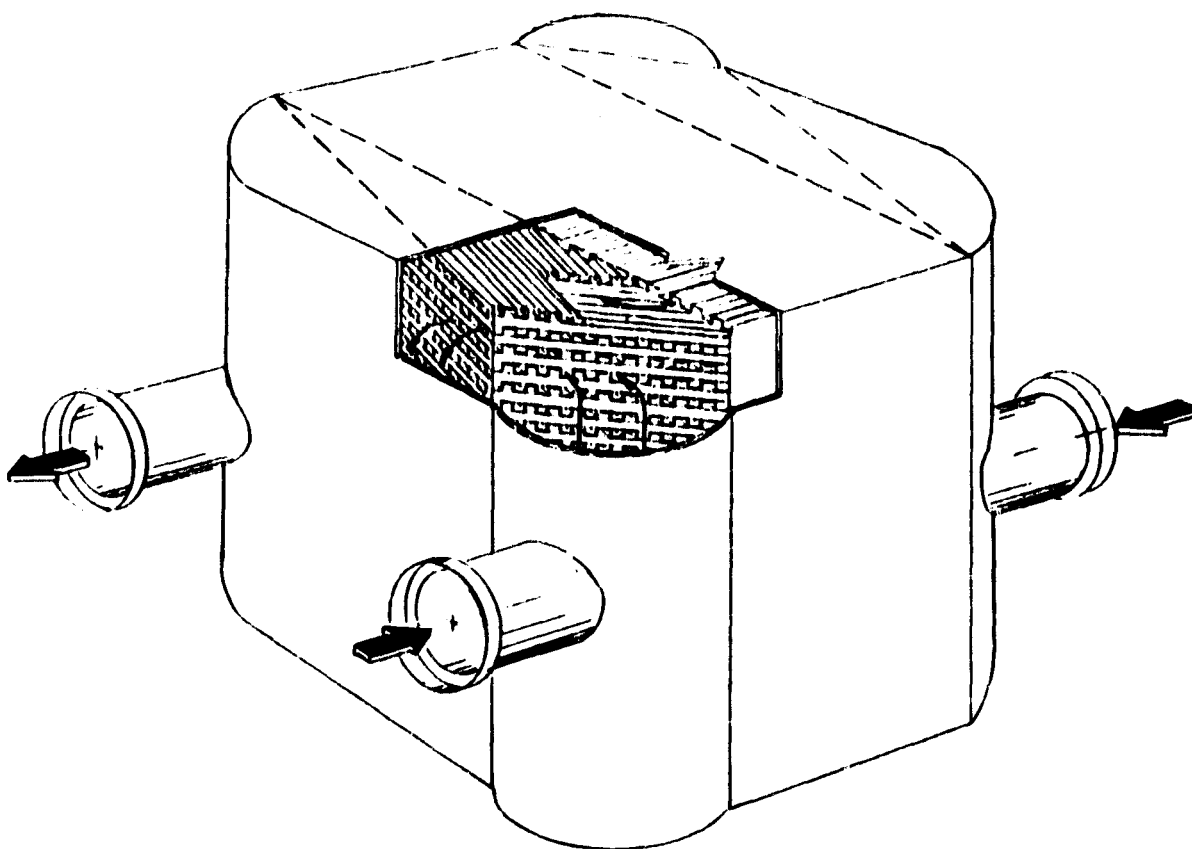


Figure 26. Recuperator Design Concept and Characteristics





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gas properties, state point, and desired effectiveness and pressure drop to calculate the geometry and mass of this heat exchanger.

Each recuperator has a mass of 365 kg (805 lb). The effectiveness of 88 percent is easily within the state of the art. (Effectiveness defines the percentage of heat available from the turbine exhaust gas which is transferred to the compressor discharge gas.) The design is compact, with dimensions shown on Figure 26. The only area needing development is manufacturing technology. Analytical study is needed to define the assembly method. The refractory materials are well-characterized, but some work is needed to demonstrate joining techniques. Other heat exchange/manufacturing options are available and should be explored.

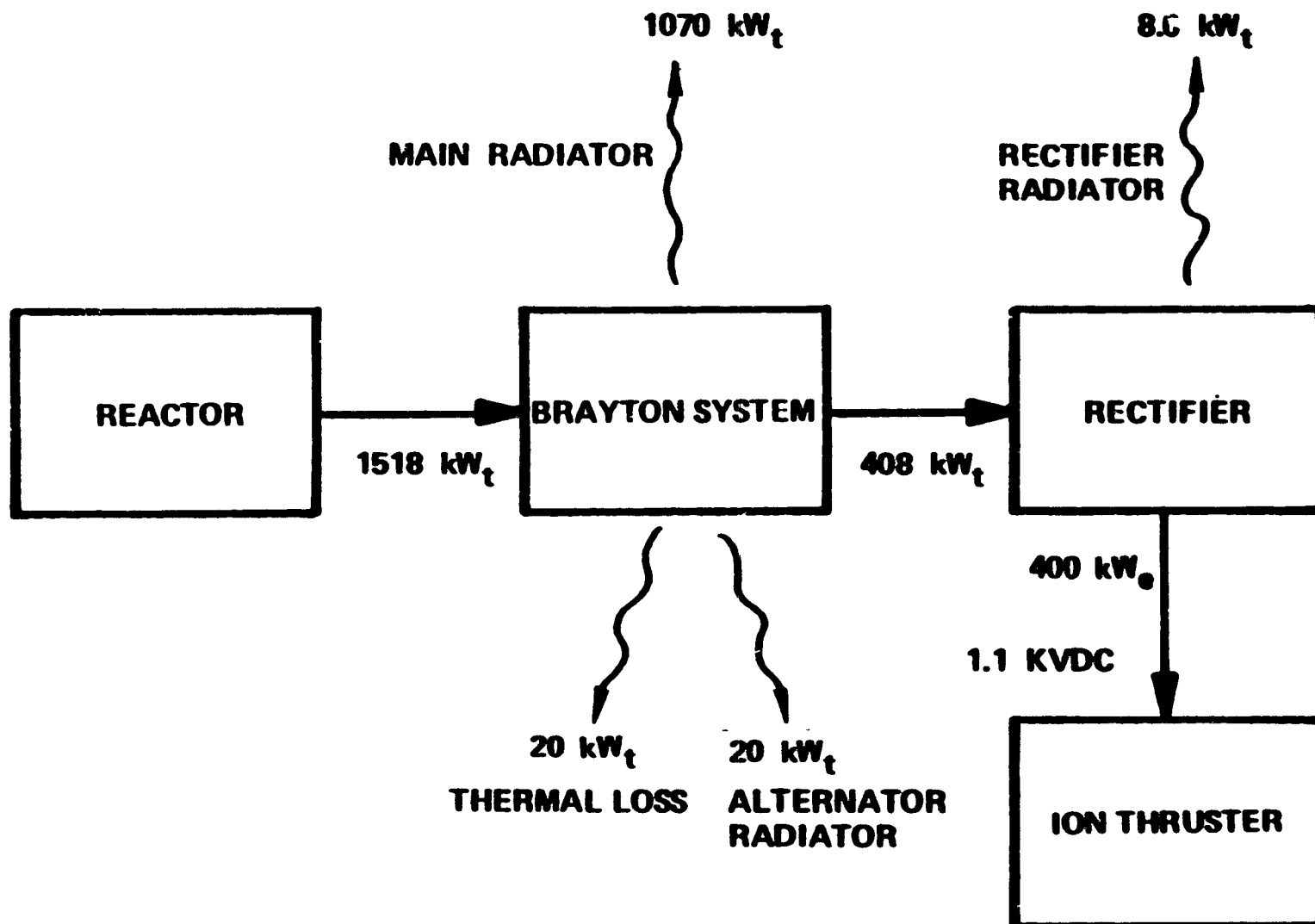
#### 2.3.6 Heat-Pipe Radiator Design

The heat-pipe radiator is the largest and most massive component in the power system. This radiator rejects the fraction of the input heat that is not converted to electricity to the sink of space. Figure 27 illustrates the heat balance for the entire system. The geometry of the radiator is defined by the closed cycle engine design program. This geometry is a function of heat rejection rate, compressor inlet temperature, pressure drop, and flow rate. This radiator was designed to fit into the Space Shuttle bay.

The cylindrical radiator is composed of eight identical panels. The amount of bending of the heat pipes required to yield the cylindrical geometry has no deleterious effect on the heat-pipe performance, according to Thermacore. The manner in which these panels overlap is shown in Figure 28. The gas heat exchanger of each radiator is protected from micrometeoroid penetration by the overlapping heat pipes from the adjacent panel. A multifoil insulation blanket is also sandwiched between the gas heat exchanger and heat rejection panel to provide thermal isolation and additional micrometeoroid protection.



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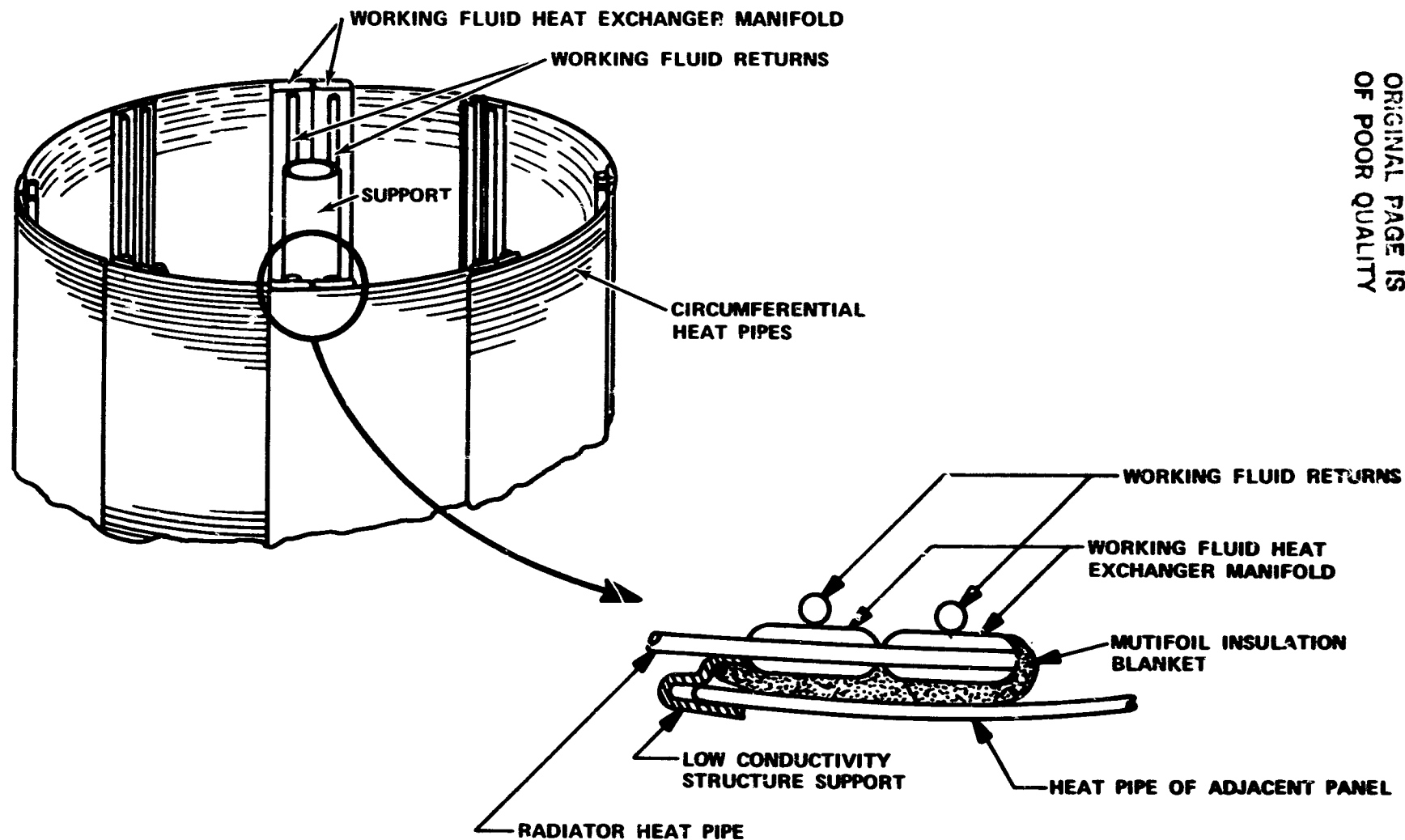
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Figure 27. Heat Balance for 400-kW<sub>e</sub> Brayton Power System.

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Figure 28. Cylindrical Heat Pipe Radiator Conceptual Design



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Figure 29 is an illustration of the heat-pipe radiator that shows the gas heat exchanger, heat pipes, and fin. There are two gas heat exchangers, either of which can carry the entire thermal load to the heat pipes. When both redundant power systems are operated at half power, each heat exchanger carries half of the thermal rejection load. In any event, the radiator panel always carries the entire thermal load.

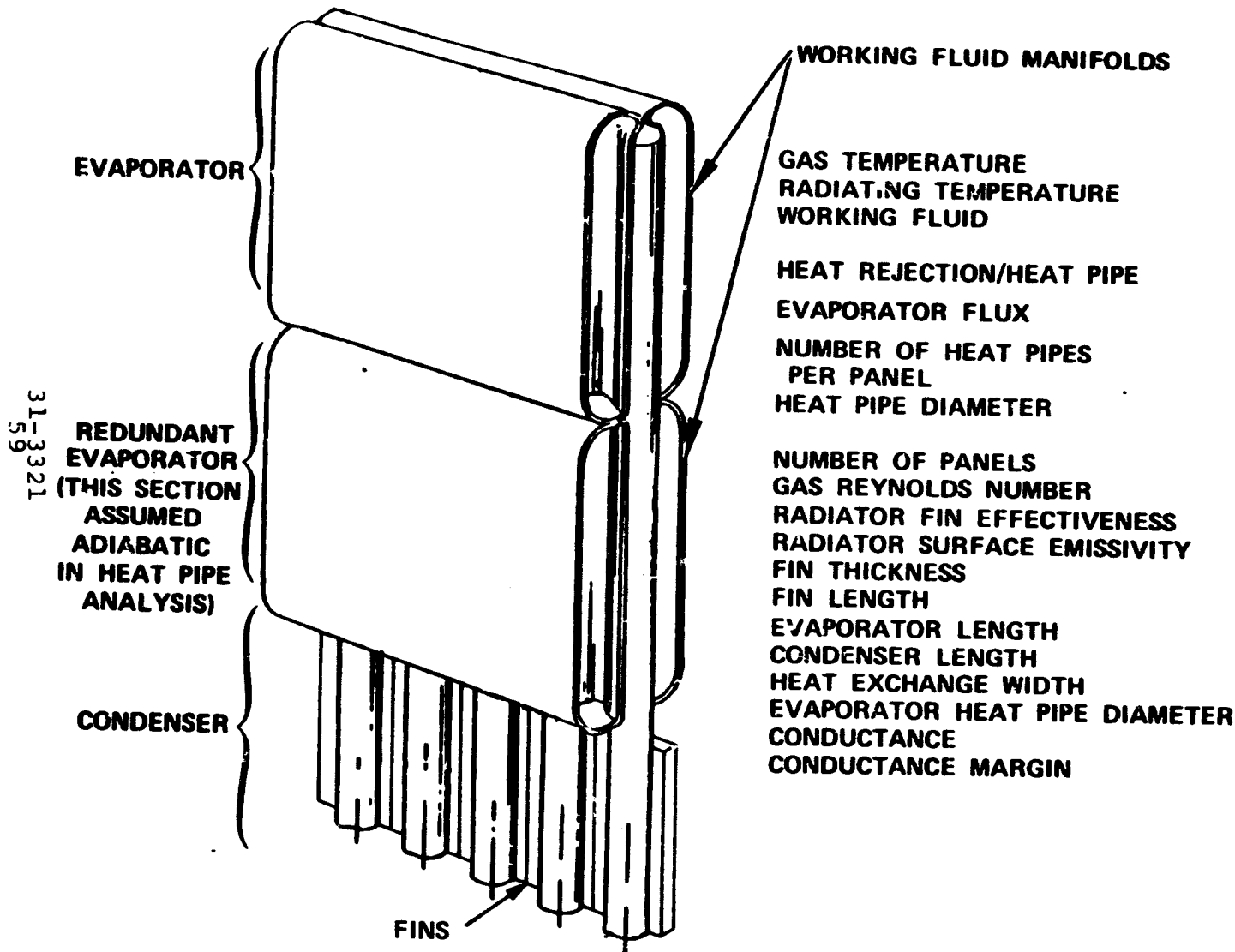
Figure 29 also lists the performance of the hot-end and cold-end heat pipes as well as the geometry of the gas heat exchanger. Diameter of the evaporator heat pipes is 2.5 cm (1 in.) to achieve adequate gas-side heat transfer area. The diameter of condenser heat pipes varies from 0.6 to 1.3 cm. This use of dual diameters greatly reduces the mass of armor from that which otherwise would have been required. The mass of the heat pipes is also reduced.

As the data in Appendix A show, the smaller diameter heat pipes have adequate heat-transfer capacity with reasonable temperature drop down the pipe. Clearly, a larger diameter heat pipe would have a greater heat transfer capacity. While analytical methods have not been fully characterized for a tapered pipe (e.g., a large diameter evaporator coupled to a smaller diameter condenser with a short tapered transition), such pipes have been fabricated and tested. Thermacore has concurred that this approach is basically sound and that there should be very little uncertainty of the feasibility of such a pipe.

A mass summary for the radiator is given in Table 7. The two largest contributors are the gas heat exchanger and heat pipes. Some mass reduction in the heat pipes is possible if thinner walls are used (the present design has a wall thickness of 3 to 6 percent of diameter). Mass reduction is also possible with the new armor design being investigated by Thermacore. Furthermore, the mass of the gas heat exchanger could be reduced by use of composite materials. Some



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GAS TEMPERATURE  
RADIATING TEMPERATURE  
WORKING FLUID

HEAT REJECTION/HEAT PIPE  
EVAPORATOR FLUX

NUMBER OF HEAT PIPES  
PER PANEL  
HEAT PIPE DIAMETER

NUMBER OF PANELS  
GAS REYNOLDS NUMBER  
RADIATOR FIN EFFECTIVENESS  
RADIATOR SURFACE EMISSIVITY  
FIN THICKNESS  
FIN LENGTH  
EVAPORATOR LENGTH  
CONDENSER LENGTH  
HEAT EXCHANGE WIDTH  
EVAPORATOR HEAT PIPE DIAMETER  
CONDUCTANCE  
CONDUCTANCE MARGIN

**HOT END**

770K  
691K  
RUBIDIUM OR  
MERCURY  
1.0 kW<sub>t</sub>  
5 W<sub>t</sub>/cm<sup>2</sup>

**COLD END**

500K  
484K  
DOWTHERM A  
0.23 kW<sub>t</sub>  
1 W<sub>t</sub>/cm<sup>2</sup>  
105  
1.3 cm

8  
85700  
0.98  
0.90  
0.25 cm  
1.25 cm  
27.5 cm  
16.0 cm  
5.6 cm  
2.5 cm  
3.8 kW<sub>t</sub>/K  
20%

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**Figure 29. Heat Pipe Radiator Heat Exchanger Design Concept and Characteristics.**



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TABLE 7

## MASS SUMMARY FOR RADIATOR WITH DUAL-DIAMETER HEAT PIPES

<u>Component</u>	<u>Mass, kg</u>
Evaporator Heat Pipes	766
Heat Exchanger Wrap Up	1433
Condenser Heat Pipes	650
Armor	439
Fins	752
	<hr/>
Total	4040



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slight reduction in fin mass is also possible. Summing these contributions would result in a total radiator mass of less than 3800 kg. This would reduce the system specific mass (Jovian environment) to 20 kg/kW<sub>e</sub>.

Stainless steel was selected for the heat pipes because of its compatibility with the heat pipe working fluids. Steel was also selected for the gas heat exchanger because of its strength to weight ratio. Beryllium or Lockalloy (Be38Al) was selected for the armor because of its low density, high modulus of elasticity and high thermal conductivity. Either of these materials could also be used for the fin because of low density and high conductivity.

#### 2.3.7 Power Conditioning and Associated Heat Rejection

The power conditioning is accomplished by a solid state device which converts 500 VAC line to neutral to 1100 VDC. This device is basically a bridge rectifier. Efficiency is 98 percent.

The waste heat from this device must be rejected to space. A separate heat-pipe radiator is used. The evaporator is thermally attached to the appropriate electronics. The condenser is attached to a fin and radiates the waste heat to space. For the 400-kW<sub>e</sub> NEP, this radiator is a right circular cylinder 0.6 m high and 4 m in diameter. During launch, this radiator is stored inside main radiator with the payload. After launch, the radiator and payload are deployed, allowing the closed cycle engines and thrusters to be activated.

#### 2.3.8 Alternator Radiator

As shown in Figure 27, the alternator radiator must remove 20 kW<sub>t</sub>. This radiator comprises the aft 2 m of the 3.2 m diameter cylindrical enclosure of the power system which is sufficient to maintain the alternator temperature at 390°K (240°F). Contact with ion engine designers indicates that there will be minimal interaction with ion beam and that thin graphite coatings ( $\epsilon \approx 0.85$ ) will provide satisfactory erosion resistance.



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### 3.0 CONCLUSIONS

Current results clearly show that a nuclear electric spacecraft with a 400-kW<sub>e</sub> reactor Brayton power system can be placed in orbit by a single Shuttle launch. The 400-kW<sub>e</sub> reference system identified by this study employs a 1990 technology turbine inlet temperature of 1500°K (2240°F) using available refractory alloys and other conservative design parameters. Practical systems over the power range from 100 to 1000 kW<sub>e</sub> have been identified with TITs from 1325 to 1650°K, and possibly 1800°K. Essentially current (1985) to projected (1995) technologies are represented, utilizing materials from superalloys to high-temperature refractory alloys and, ultimately, to ceramics. Use of a heat-pipe-cooled reactor, independent redundant power conversion loops, and heat-pipe radiators eliminates the single-point failure modes. The specific mass performance parameter for the 400-kW<sub>e</sub> power level is within 5 percent of the goal of 20 kg/kW<sub>e</sub> which compares favorably with power systems based on other conversion means. Further refinement of the 400-kW<sub>e</sub> system design with the new LASL layered core reactor, specifically tailored Brayton components, and more detailed design of the primary radiator based on new heat-pipe concepts will provide specific system masses below 20 kg/kW<sub>e</sub>.

The heat-pipe cooled reactors now being defined by LASL utilize a relatively new concept that employs advanced technology. These reactors have great promise for broad applicability. Substantial research and technology work as well as subsequent development and testing are needed and should be strongly supported.

While closed Brayton cycle technology is receiving support because of its breadth of recognized applications, the special requirements for space service (e.g., performance parameters such as low specific mass, high efficiency, and reasonable specific radiator area; very high reliability with long life; and low comparative costs) need to be evaluated by parametric systems analysis and subsequent





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research and technology work. Typical topics that need to be addressed include high-temperature materials, advanced heat exchangers, rotating machine elements including bearings, heat-pipe space radiator concepts, power processing elements, etc. Broad research and technology efforts in applicable types of advanced heat pipes are especially needed.

Future space missions are currently not well enough defined to permit proper focusing of technology and development efforts. More detailed optimizations of various classes of advanced missions are needed combining spaceflight trajectory analysis and parametric systems analysis with programmatic factors in the 1990 to 2010 time frame and beyond. This work is needed to help identify the kinds of technology that will be most useful and its timing.

Based on the study results, space power systems that use closed Brayton cycles may well find application in future nuclear electric spacecraft when definitive comparisons with other systems that include all pertinent factors are completed. Clearly, further work is needed to ascertain the degree and extent of the promise of CBC system vis-a-vis the competing power conversion systems.



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#### 4.0 RECOMMENDED FURTHER STUDIES AND ANALYSES

Based on results of the work accomplished in this study as well as previous efforts over many years on space power system development, the following recommendations are made for further studies and analyses of reactor Brayton power systems for nuclear electric spacecraft. In order to keep a technology viable, some continuity of effort is needed that keeps a minimum level of effort active. This is especially true at present when the new capabilities of the Space Shuttle transportation system are beginning to be understood and exploited. The Shuttle will have an increasing influence on the course of solar system exploration in the late 1980s, 1990s, and beyond, especially as the contribution of nuclear electric rocket propulsion is realized. Closed Brayton cycle power conversion, in both its established and advanced forms, has a role to play in the space power systems that are needed for solar system exploration, as well as a wide variety of other missions.

##### 4.1 Refined System Designs, Including Reliability-Life Characteristics

The conceptual design of the 400-kW<sub>e</sub> reference system should be refined to include LASL's new layered core reactors, specifically designed Brayton components, and detailed design of space radiators based on well-developed heat-pipe data. It may be desirable to investigate other specific power levels; for example, the currently identified 1200-kW<sub>t</sub> LASL layered core reactor would provide between 250 and 350 kW<sub>e</sub> using a Brayton power conversion system. In addition to determination of performance parameters in greater detail, it is timely to undertake the introduction of reliability and life factors in the analysis. The conversion of the present system design code into a parametric systems analysis code with modular elements should also be investigated.



#### 4.2 Advanced Component Design

Current analytical techniques do not allow tailoring each Brayton component in detail for specific mission as well as system requirements. AiResearch recommends that the effect of novel heat transfer concepts, materials, configurations, and other requirements be investigated for the major Brayton components. A typical system design, such as the current 400-kW<sub>e</sub> approach, would form the basis for this effort. Such important considerations as reliability, life, and cost need to be included as well as performance tradeoffs.

#### 4.3 Space Radiator Designs with Advanced Heat Pipes

The primary heat rejection radiator is the dominant element in Brayton power systems for space applications. Special attention should be given to further conceptual design and analysis of space radiators with advanced heat pipes configured especially for this use. Many neoteric heat pipe designs, with wall materials and working fluids matched to the specific requirements of temperature, shape, meteoroid protection, and other pertinent requirements, should be evaluated.

#### 4.4 System Operations and Safety

A study is recommended of the operational requirements of Brayton systems in space power service. The various operational phases such as system assembly, ground checkout, launch operations, orbital transfer, automated start-up, and power system regulation in space need to be studied and analyzed, especially with respect to their effect on overall system safety. Interactions of the Brayton components with the nuclear subsystem, spacecraft, and space transportation systems should be evaluated in, at least, an introductory manner. Preliminary outlines of the required system safety documents should be generated for interactive discussions with cognizant agencies.



#### 4.5 Probability of Mission Success

The influence of the power system on the probability of mission success and the design and operational characteristics of the Brayton components are recommended for study in detail. This will require definition of typical missions in terms of mission goals and operational sequences. Since nuclear electric spacecraft will be used for complex missions throughout the solar system for durations of ten or more years, these reliability/availability studies are required to maximize the probability of performing a wide variety of missions successfully.

#### 4.6 Economic Analyses Including Risks

Although non-recurring and recurring costs of Brayton power systems for space cannot be determined now with satisfactory accuracy (since only conceptual system designs and sketchy mission application information are available), it is not too soon to begin to develop the methodology for economic analysis. At the least, this will result in the necessary data being identified. Economic analyses of advanced systems, especially if they are to deal with risk, must be conducted on a probabilistic basis. Since it is necessary to keep such analyses as simple and clearly defined as possible, as appropriate to the available data and sophistication of the results, it is strongly recommended that the methodology for such analyses be formulated relatively early in the development program.

#### 4.7 Space Shuttle Compatibility

An understanding of the Space Shuttle transportation system in some detail is necessary to be sure the Brayton-powered, nuclear electric spacecraft will be compatible with ground, launch, and space operations. Early attention to the detailed interfacing of the Brayton power system with the nuclear electric spacecraft and the Space Shuttle orbiter in all regimes is recommended.



## 5.0 RECOMMENDED TECHNOLOGY EFFORTS

In general, the closed Brayton power conversion system can be characterized as a mature technology ready for application (e.g., flight system development). The rather unusual operating conditions\* required for the optimal systems defined in this study engenders a requirement for technology development in certain areas. Also, it must be recognized that there is currently very little supporting research or advanced technology either underway or being proposed which is aimed specifically at the requirements of space Brayton systems. Therefore, this section addresses these supporting technology needs and defines a preliminary schedule. This program would result in the required technology being demonstrated by target dates that are commensurate with current projections of flight system applications.

As summarized in Figure 30, a scenario of projected technology utilization for prospective flight applications has essentially been developed in Section 2.0 of this report. This figure indicates that the power requirements will increase exponentially with time. As described previously, a five-year flight system development and qualification program is assumed; therefore, technology readiness\*\* needs to have been demonstrated five years before the proposed launch date.

\*In terms of past space studies as well as more contemporary analyses of terrestrial power systems, the results of this study are characterized by lower efficiencies, higher ratios of CIT to TIT and lower recuperator effectivenesses to minimize the radiator mass and, thereby, the system mass.

\*\*Technology readiness is that stage of system, subsystem, or component development where all major problems associated with achieving the specified on performance goals have been solved and where the solutions to problems have been successfully demonstrated through actual hardware design, fabrication, and test programs. At this stage, there remain no major risks for an agency or contractor in scaling up the technology (if full-scale demonstration has not been performed) and in proceeding with mission/commercial development of the system, subsystem, or component.

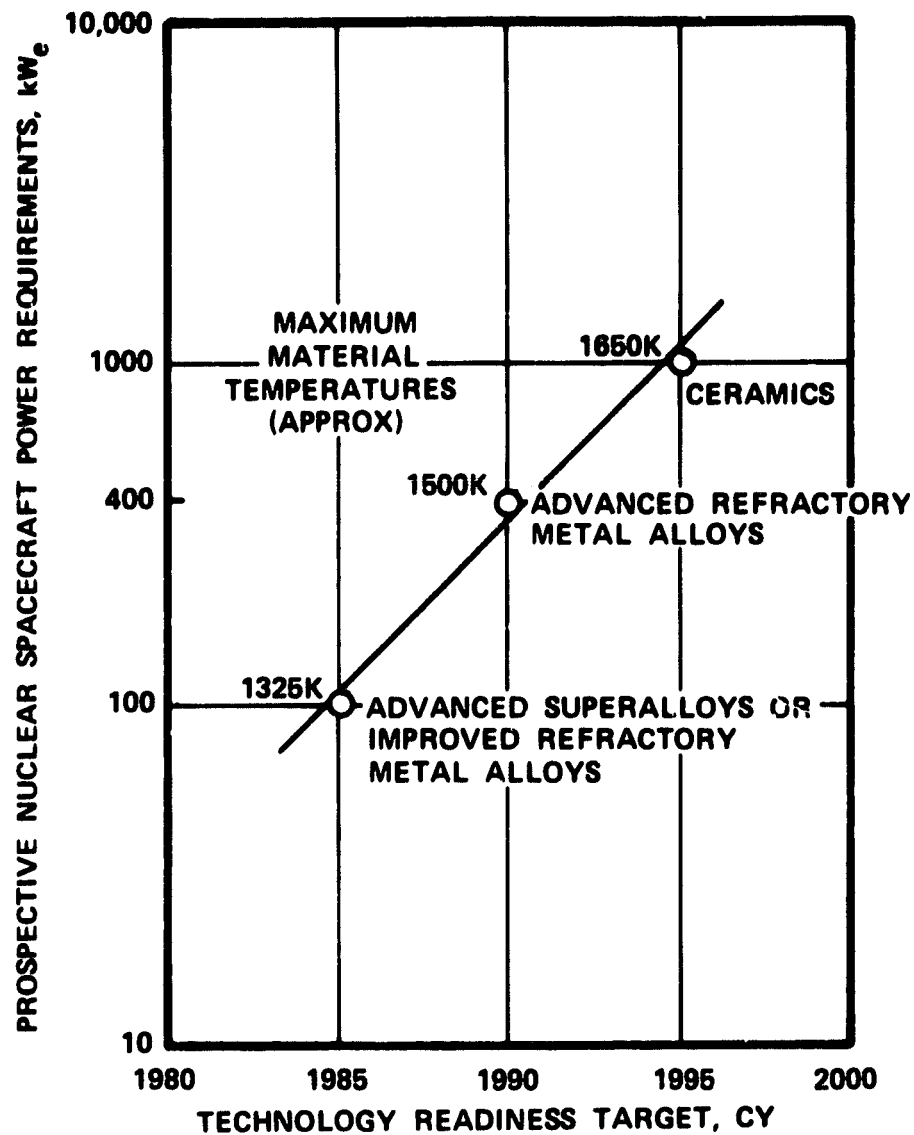


Figure 30. Prospective Nuclear Spacecraft Power Requirements Versus Technology Readiness.



Figure 30 shows three increasingly stringent levels of nuclear reactor powered system applications:

- o 100-kW<sub>e</sub> power systems in the early 1990s
- o 400-kW<sub>e</sub> power systems in the mid 1990s
- o 1000-kW<sub>e</sub> power systems at the turn of the century

A schedule which lists the technology development areas required for increasingly sophisticated missions is shown in Figure 31, and discussed in some detail subsequently. It is important to note that approaches to the required technology can be defined at this time, e.g., no fundamentally new technology or "technical breakthrough" is required.

In projecting the technology requirements, the reactor heat source is not included. It was assumed that the current LASL development program will continue under independent sponsorship and that this program will yield the reactor technology as it becomes required.

### 5.1 100 kW<sub>e</sub> System Technology

The current application forecast for this system is geocentric orbital power. Because of the large Shuttle lift capability for this class of missions, it is not necessary to achieve minimum system mass or volume to have a viable approach. This resulted in the choice of the 1325°K TIT, as noted previously.

The most necessary technology for this class of missions is in the area of materials. Current superalloys are limited to 1144 to 1172°K (1600 to 1650°F). Late developments in alloy modification using rapid solidification with powder metallurgy have currently resulted in allowable TIT increases of 56 to 83°K (100 to 150°F) with further improvement possible.

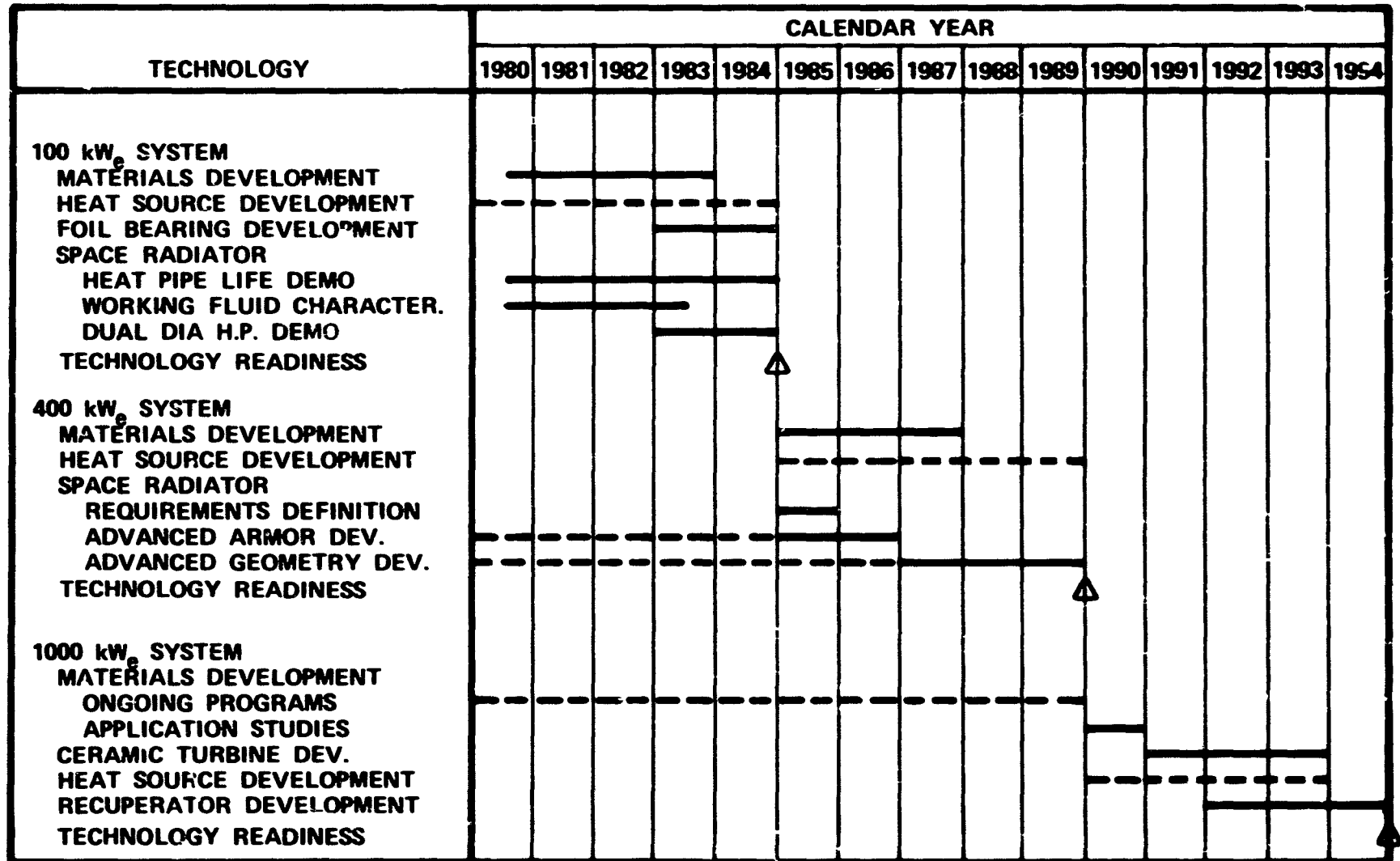


Figure 31. Schedule of Key Power Conversion Technology Elements Required for Reactor/Brayton Space Power Systems.

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The relatively well characterized refractory metals such as niobium have adequate stress carrying capabilities but questions regarding potential degradation due to even the low oxygen partial pressure which exists in near-Earth space must be resolved. For example, the durability of existing anti-oxidation coatings in the hard vacuum of space must be established.

A substantiated bank of standardized design data needs to be assembled, including definition of design nominals and minimums as well as an accepted methodology for life projection. Thus, a fairly comprehensive materials characterization effort will be required.

The reactor heat pipes are exposed to the highest temperatures in the closed loop and, as previously noted, will be made from refractory metals such as molybdenum or niobium. Methods of efficiently integrating these heat pipes into the heat source heat exchanger need to be studied, analytically and empirically. Potential approaches include an all-refractory metal design as well as one in which the heat exchanger wall could be made from metal, using internal insulation to limit hot spot exposure. In the latter case, either a thermally resilient seal between the refractory and superalloy or a weldment/brazement into a diaphragm-type thermal stress relief might be used.

In recent years, foil type gas bearings have come into increasingly wide-spread use in various commercial turbomachinery. For example, the bearings on the DC-10 air-conditioning air cycle units have accumulated over  $53 \times 10^6$  bearing operating hours of operation and currently exhibit a mean time between bearing failure of 249,000 hours. Concurrently with these applications, dramatic strides advanced this technology from an empirical base to a discipline solidly supported by analysis and field operation. However, this analytical characterization effort has recently stagnated. To provide a proper technology base, this effort needs to be re-instituted to



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establish a uniform methodology (including analytical, fabrication, inspection, and performance verification) for a variety of applications including space power systems. An important element is the development of a hydrodynamic/elastic computer model. The derivation and empirical verification of the predictions of such a model will be extremely significant in the avoidance and/or timely correction of problems during future applications.

The heat-pipe radiator for the 100-kW<sub>e</sub> system (Figure 4) requires very little extension to existing technology. A ground-based, horizontal, extended life test of heat pipes using Dowtherm A and either mercury or rubidium working fluids should be instituted as early as practical to demonstrate that no unanticipated life-limiting mechanisms will be encountered. The data base for rubidium, especially, and mercury, to a certain extent, is insufficient for the heat-pipe designer's needs and should be expanded. The thermal decomposition or other life-limiting behavior of Dowtherm A should be investigated thoroughly as a function of temperature. To cut down the radiator mass, a statistically significant number of dual diameter (large evaporator, small condenser) heat pipes should be fabricated and subjected to thermal performance limit testing. Space testing of representative heat pipe designs should be undertaken at the earliest practical opportunity.

## 5.2 400-kW<sub>e</sub> System Technology

Since the interplanetary spacecraft propulsion system uses the full Shuttle launch capability both in terms of injected mass and payload bay volume, power system performance is critical. Therefore, a higher TIT was selected for this system. In discussing the technology development implications for support of this system, it has been assumed that the items described in the preceding paragraph will have been completed, either in support of the 100-kW<sub>e</sub> system development or under other program sponsorship in the intervening period.



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The biggest payoff for the 400-kW<sub>e</sub> system is in the area of heat-pipe space radiator technology development. A comprehensive effort should be completed to evolve the most realistic set of environmental design requirements, particularly in the definition of micrometeoroid flux levels. The variations in such exposure over the candidate missions should be defined so that the armor is designed realistically.

Armor geometries, materials, and manufacturing and application techniques should be investigated to minimize the penalty due this factor. The results of the current Thermacore study suggest that this topic has only recently begun to be pursued in the required depth. However, it may well have been completed prior to 1985, as the dashed line in Figure 31 indicates.

A concomitant effort on the investigation of the performance improvements possible by using advanced geometry heat pipes should also be completed, if not accomplished in the intervening years. Configuration-pumped and mechanically-pumped heat pipes are examples of the advanced types that need to be considered.

Additional materials development will be required to enable the 1500°K TIT (Figure 18) to be attained. Molybdenum and tantalum alloys are the leading candidates. The data base on these materials is sketchy and quite dated; thus, a significant effort in this area is indicated. Machinability, manufacturability, joining, and space environment tolerance under operating conditions are additional topics that will need to be addressed.

It will also be necessary to integrate the more advanced refractory materials into the heat source heat exchanger. It is believed that techniques which will have been developed for the 100-kW<sub>e</sub> heat source will be applicable, but this needs to be confirmed.



### 5.3 1000-kW<sub>e</sub> System Technology

The projected missions and launch vehicles for this class of power systems are in the very early definition phase. Thus, it cannot be stated unequivocally that the higher performance selected for this system will absolutely be required. In the eventuality that lower performance is acceptable, the 1000-kW<sub>e</sub> missions could be accomplished by ganging three of the 400-kW<sub>e</sub> systems with a fourth as the redundant spare to yield an overall system with fairly respectable performance specifics.

However, it must be recognized that the historical trend in aerospace is to use the available technological capability to the fullest. Few rocket-powered vehicles have been launched that used only a fraction of their lifting capability, and there is little reason to suspect that this trend will change in the future (early projections regarding the usage of the space transportation system notwithstanding). Therefore, this section will address technology development required to support the CBC systems operating at a TIT of 1650°K (2510°F) as listed in Figure 30.

In common with the systems described previously, the largest implication is on the materials needed for the high temperature components. As noted in Section 2.1.4 and indicated by the dashed line in Figure 31, the ceramic technology needed for this system is currently under active development for a number of programs. The differing requirements of the space power system will necessitate that a thorough analytical study, with possibly some supporting empirical effort, be undertaken to define how this technology may best be applied.

Following this effort, three fabrication/demonstration efforts associated with materials integration into critical components should be pursued. The ceramic turbine and associated static components



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should be fabricated and assembled unless prior efforts have shown that components of the appropriate scale and materials have already been achieved. Most probably, ceramics will have to be used extensively in the reactor and ceramic heat pipes will be used between the reactor and heat source heat exchanger. The approach for providing extended surface on the condenser end in the heat source heat exchanger and the methods for sealing and integrating the heat pipes through the wall (ceramic or metallic) of the heat exchanger must be developed and demonstrated. Higher temperature capability materials are also needed for the recuperator. However, this should be within the range of the advanced refractories that will have been demonstrated at that point. Therefore, a fabrication/assembly technology demonstration program with these materials should be accomplished.



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**APPENDIX A**

**PRELIMINARY LETTER REPORT #1  
FROM  
THERMACORE, INC.**

**(56 Pages)**



**THERMACORE, INC.**

LEOLA, PENNSYLVANIA

April 1979

PRELIMINARY LETTER  
REPORT  
#1

CONTRACT 955437

METEOROID PROTECTION METHODS  
FOR SPACECRAFT RADIATORS  
USING HEAT PIPES

PREPARED FOR

CALIFORNIA INSTITUTE OF TECHNOLOGY  
JET PROPULSION LABORATORY  
PASADENA, CALIFORNIA

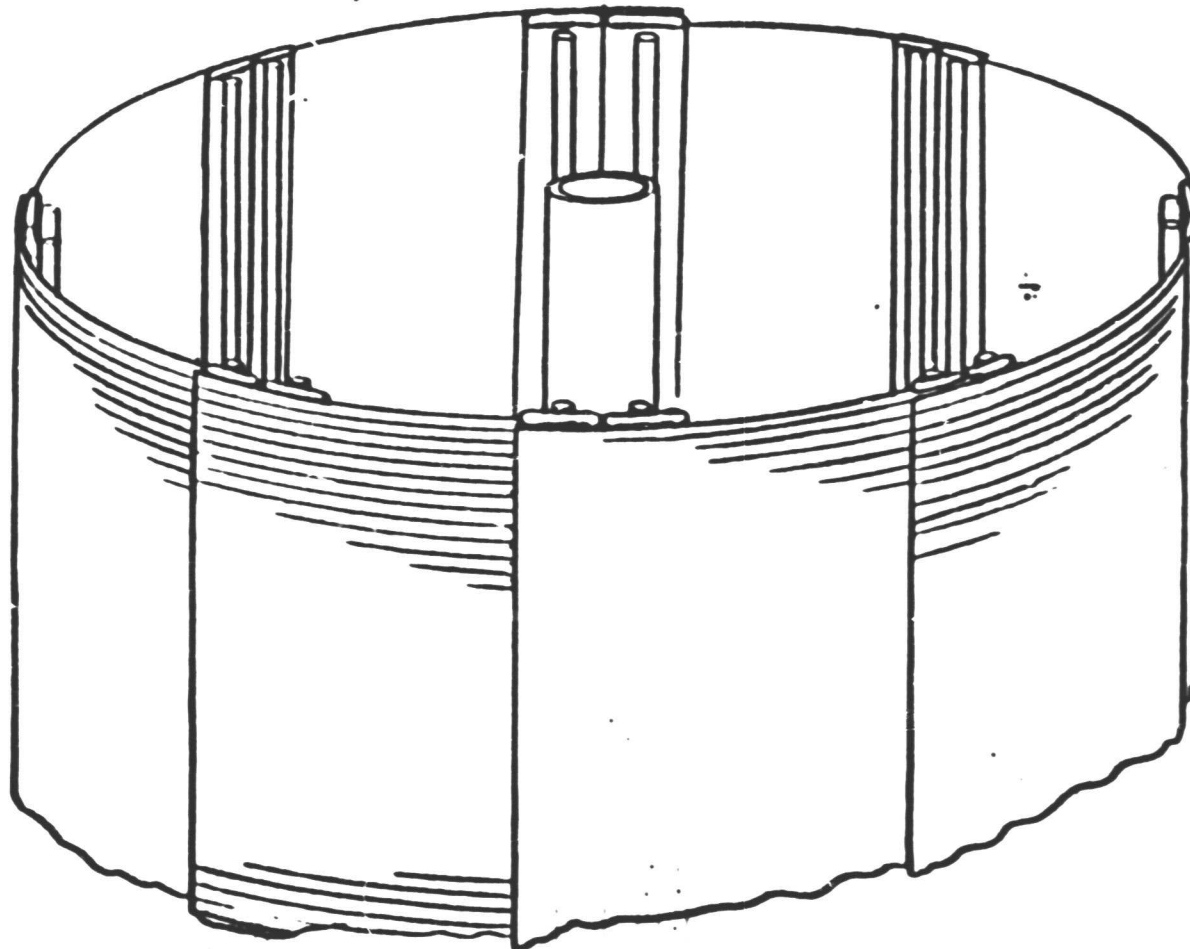
## HEAT PIPE DESIGN FOR CBC RADIATOR

The 400 kW<sub>e</sub> Closed Brayton Cycle power system for the Nuclear Electric Propulsion Spacecraft has been designed by Garrett AiResearch<sup>2</sup> to use heat pipes to achieve a thermally effective radiator which has a high survival probability. It is also anticipated that the heat pipe design will lead to a low specific mass. The heat pipe design evaluated in this work is for use in a cylindrical array as seen in Figure 3.1. This design has eight dual gas-to-radiator heat pipe heat exchangers fed from a dual central duct. The heat pipes are attached to both gas ducts over a length of 43 cm on each duct. Thus, the heat pipes provide armor protection for the gas ducts.

In normal operation, the total 86 cm length attachment over the heat pipes to the gas ducts will be used as heat pipe evaporators. The condenser is 176 cm long. If either gas duct or engine fail, then the whole power load will be transferred to the heat pipes through only one of the 43 cm attachments. Accordingly, for design consideration, the heat pipe must be sized as though it had a 43 cm evaporator, 43 cm adiabatic and 176 cm condenser.

Four different sets of heat pipe designs were analyzed with respect to mass and performance. However, no consideration was given to the required heat pipe armor and tradeoffs in the heat pipe diameter versus T-bar fins for total mass. The overall heat pipe cell dimension as designed by GAR is 3.175 cm (1.25") and includes heat pipe and fins. All heat pipes discussed in the Sections 3.1 and 3.2 have computer printouts of their performance tabulated in Appendix 1.

FIGURE 3.1



CBC RADIATOR CONFIGURATION

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### 3.1 Baseline Design

The total power to be dissipated is  $1.1 \times 10^6$  watts. From the gas side of the radiator heat exchanger, heat pipe temperatures were calculated by Garrett AiResearch to range from  $707^\circ\text{K}$  down to  $492^\circ\text{K}$ . The power levels are 720 watts per heat pipe at  $707^\circ\text{K}$  and 169 watts per heat pipe at  $492^\circ\text{K}$ . Thus,  $\sigma A \epsilon$  can be computed to be  $2882 \times 10^{-12}$  watts/ $^\circ\text{K}^4$  from:

$$P = \sigma A \epsilon T^4 \quad \text{Equation 3.1}$$

where

$P$  = power radiated - watts

$\sigma$  = Stefan Boltzman Constant =  $5.67 \times 10^{-12} \frac{\text{watts}}{\text{cm}^2 - ^\circ\text{K}^4}$

$T$  = heat pipe temperature -  $^\circ\text{K}$

$A$  = individual heat pipe radiating area -  $\text{cm}^2$

$\epsilon$  = effective thermal emissivity

Table 3.1 shows the required heat pipe power for each of the end temperatures and each temperature divisible by  $25^\circ\text{K}$ .

Garrett AiResearch's baseline design is a 2.54 cm (1") O.D. heat pipe with a 0.0762 cm (.03") wall. The optimum heat pipe designs under these conditions are seen in Table 3.2. Rubidium is the preferred heat pipe fluid from  $707^\circ\text{K}$  down to  $650^\circ\text{K}$ . Below  $650^\circ\text{K}$  Dowtherm A (DTA) is the preferred fluid. In both cases, a screen covered groove design is found to be the lowest mass system. The rubidium heat pipes have a 1.75 Kg mass. The DTA heat pipes have a 1.74 Kg mass.

Table 3.3 shows the same heat pipes, which have been, for the most part, optimized with respect to the number of grooves and their aspect ratio. The rubidium heat pipes have a 1.48 Kg mass. The DTA heat pipes have a 1.55 Kg mass.

TABLE 3.1

Temperature		Req. Power
<sup>o</sup> K	<sup>o</sup> C	Watts
707	434	720
700	427	692
675	402	598
650	377	514
625	352	440
600	327	373
575	302	315
550	277	264
525	252	219
500	227	180
492	219	169

REQUIRED POWER PER HEAT PIPE AT ELEVEN  
DIFFERENT TEMPERATURES

TABLE 3.2

## HEAT PIPE MASS &amp; PERFORMANCE FOR BASELINE DESIGNS

Evaporator - 43 cm			Fluid: Rb Vessel: 304 SS				Fluid: DTA Vessel: 304 SS			
Adiabatic - 43 cm			O.D.: 2.54 cm Wall: 0.0762 cm				O.D.: 2.54 cm Wall: 0.0762 cm			
Condenser - 176 cm			# Grooves: 25 Groove Width: 0.2 cm				# Groove: 27 Groove Width:0.2 cm			
S = Sonic Limit										
C = Capillary Limit										
Req. Power			$\Delta T @$ Req. Power	Power Limit	Mass	Groove Depth	$\Delta T @$ Req. Power	Power Limit	Mass	Groove Depth
$^{\circ}K$	$^{\circ}C$	Watts	$^{\circ}C$	Watts	Kg	Cm	$^{\circ}C$	Watts	Kg	Cm
707	434	720	2.56	1750-S	1.75	0.05				
700	427	692								
675	402	598								
650	377	514	6.44	608-S	1.75	0.05				
625	352	440					3.89	545-C	1.74	0.065
600	327	373								
575	302	315								
550	277	264								
525	252	219								
500	227	180								
492	219	169					1.73	710-C	1.74	0.065

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TABLE 3.3

## OPTIMIZED HEAT PIPE MASS &amp; PERFORMANCE - BASELINE DESIGN

Evaporator - 43 cm Adiabatic - 43 cm Condenser - 176 cm S = Sonic Limit C = Capillary Limit			Fluid: Rb    Vessel: 304 SS O.D.: 2.54 cm    Wall: 0.0762 cm # Grooves: 25    Groove Width: 0.275 <sub>cm</sub>				Fluid: DTA    Vessel: 304 SS O.D.: 2.54 cm    Wall: 0.0762 cm # Groove: 25    Groove Width: 0.275 <sub>cm</sub>			
Temperature    Req. Power			$\Delta T @$ Req. Power	Power Limit	Mass	Groove Depth	$\Delta T @$ Req. Power	Power Limit	Mass	Groove Depth
<sup>o</sup> K	<sup>o</sup> C	Watts	<sup>o</sup> C	Watts	Kg	Cm	<sup>o</sup> C	Watts	Kg	Cm
707	434	720	2.43	815-C	1.48	0.02				
700	427	692								
675	402	598								
650	377	514	5.83	640-S	1.48	0.02				
625	352	440					9.32	507-C	1.55	.055
600	327	373								
575	302	315								
550	277	264								
525	252	219								
500	227	180								
492	219	169					3.55	555-C	1.55	.055

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The average mass reduction is 14%. Further optimization may result in an additional 1 or 2% mass reduction. However, far greater mass reduction can be realized by O.D. and/or wall thickness reduction.

Table 3.4 shows the 2.54 cm (1") heat pipe with a 0.025 cm (.01") wall. This wall thickness is 0.01 times the diameter and has been shown to be acceptable for use as a heat pipe containment vessel where external buckling is the ultimate constraint, i.e., the internal pressure of the heat pipe was less than 14.7 psi, thus long term creep due to hoop stress was low.

The use of a wall thickness 0.01 times the diameter was developed for Niobium, which has a modulus of elasticity of  $15 \times 10^6$  psi. This includes a safety factor of 2. Stainless steels have moduli of about  $28 \times 10^6$  psi which reduces the thickness/diameter ratio to about 0.008 with a safety factor of 2. However, the use of 0.01 as a thickness to diameter ratio will be used to assure success.

Examination of DTA at  $625^\circ\text{K}$  shows a fluid pressure of 85 psi which develops a hoop stress of 4250 psi. This stress is acceptable, since 316 SS will only creep 0.1% in  $10^5$  hours at  $1100^\circ\text{F}$  under a stress of 6000 psi.

The rubidium heat pipes have a mass of 0.69 Kg and the DTA heat pipes have a mass of 0.78 Kg.

### 3.2 Design Optimization

Examination of Tables 3.2, 3.3 and 3.4 reveals that a reduction in diameter of the rubidium heat pipes would soon result in the heat pipe going sonic. However, the DTA pipes are capillary limited, thus a reduction in O.D. is possible. Accordingly, a higher pressure fluid, mercury,



TABLE 3.4

## OPTIMIZED HEAT PIPE MASS &amp; PERFORMANCE FOR THIN WALLED BASELINE DESIGN

Evaporator - 43 cm Adiabatic - 43 cm Condenser - 176 cm S = Sonic Limit C = Capillary Limit			Fluid: Rb      Vessel: 304 SS O.D.: 2.54 cm      Wall: 0.0254 cm # Grooves: 25      Groove Width: 0.275 cm				Fluid: DTA      Vessel: 204 SS O.D.: 2.54 cm      Wall: 0.0254 cm # Groove: 25      Groove Width: 0.275 cm			
Temperature      Req. Power			$\Delta T$ @ Req. Power	Power Limit	Mass	Groove Depth	$\Delta T$ @ Req. Power	Power Limit	Mass	Groove Depth
$^{\circ}K$	$^{\circ}C$	Watts	$^{\circ}C$	Watts	Kg	Cm	$^{\circ}C$	Watts	Kg	Cm
707	434	720	1.55	820-C	0.69	0.02				
700	427	692								
675	402	598								
650	377	514	4.56	705-C	0.69	0.02				
625	352	440					5.72	555-C	0.78	0.055
600	327	373								
575	302	315								
550	277	264								
525	252	219								
500	227	180								
492	219	169					2.49	710-C	0.78	0.055

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was used in small diameter pipes in place of rubidium. These results are seen in Table 3.5.

The mercury heat pipes are 0.635 cm (.250") in diameter with a wall to diameter ratio of 0.01. The mass of the mercury heat pipes are 0.45 Kg and have a hoop stress of 625 psi at 707°K.

The DTA heat pipes are 0.9525 cm (.37") in diameter with a wall to diameter ratio of 0.01. They have 12 grooves 0.275 cm wide by a depth that varies from 0.075 cm down to 0.05 cm. Accordingly, their mass varies from 0.31 Kg down to 0.27 Kg. The DTA heat pipes at 625°K will have a hoop stress of 1600 psi.

The mercury heat pipes of Table 3.5 have eight grooves 0.2 cm wide by 0.02 cm deep. Optimizing the number of 0.275 cm wide by .02 cm deep grooves for different power levels results in a reduction in mass. At 707°K, a five-groove heat pipe has a mass of 0.29 Kg. At 675°K, four grooves have a mass of 0.28 Kg and at 550°K, three grooves have a mass of 0.27 Kg. These results are seen in Table 3.6. Also shown in Table 3.6 is the thermal performance of two of the mercury heat pipes with 86 cm evaporators, which shows an increase in maximum power capability and a reduction in total  $\Delta T$ .

Both the DTA heat pipes of Table 3.5 and the mercury heat pipes of Table 3.6 have a performance  $\Delta T$ . Accordingly, one asks what does a  $\Delta T$  in the heat pipe mean in increased mass (length of condenser) to be able to radiate the required power? Appendix 2 develops Equation 3.2 which is the increase in mass of heat pipe due to its  $\Delta T$ .

$$dm = m \frac{l_c}{l_t} [(T_o/T)^4 - 1] \quad \text{Equation 3.2}$$

TABLE 3.5

## OPTIMIZED HEAT PIPE MASS &amp; PERFORMANCE - ALTERNATE DESIGN

Evaporator - 43 cm Adiabatic - 43 cm Condenser - 176 cm S = Sonic Limit C = Capillary Limit			Fluid: Hg Vessel: 304 SS O.D.: 0.2952 cm Wall: .01 cm # Grooves: 8 Groove Width: 0.200 cm				Fluid: DTA Vessel: 304 SS O.D.: 1.27 cm Wall: 0.0127 cm # Groove: 12 Groove Width: 0.275 cm				
Temperature			Req. Power	ΔT @ Req. Power	Power Limit	Mass	Groove Depth	ΔT @ Req. Power	Power Limit	Mass	Groove Depth
°K	°C	Watts	°C	Watts	Kg	Cm	°C	Watts	Kg	Cm	
707	434	720	2.69	930-S	0.45	0.02					
700	427	692									
675	402	598									
650	377	514									
625	352	440	1.98	900-C	0.45	0.02	15.04	515-C	0.31	0.075	
600	327	373					10.73	420-C	0.29	0.065	
575	302	315					8.38	370-C	0.29	0.06	
550	277	264	2.08	805-C	0.45	0.02	6.49	305-C	0.28	0.055	
525	252	219					4.98	240-C	0.27	0.05	
500	227	180									
492	219	169					4.05	215-C	0.27	0.05	

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TABLE 6

## OPTIMIZED HEAT PIPE MASS &amp; PERFORMANCE - ALTERNATE DESIGN

Evaporator - 43 cm Adiabatic - 43 cm Condenser - 176 cm S = Sonic Limit C = Capillary Limit			Fluid: Hg Vessel: 304 SS O.D.: 0.635 cm Wall: 0.00635 cm # Grooves: 3-5 Groove Width: 0.2 cm			Fluid: Hg Vessel: 304 SS O.D.: 0.635 cm Wall: 0.00635 cm # Grooves: Groove Width: 0.2 cm Evaporator: 86 cm Condenser: 176 cm				
Temperature		Req. Power	$\Delta T$ @ Req. Power	Power Limit	Mass	Groove Depth	$\Delta T$ @ Req. Power	Power Limit	Mass	Groove Depth
$^{\circ}\text{K}$	$^{\circ}\text{C}$	Watts	$^{\circ}\text{C}$	Watts	Kg	Cm	$^{\circ}\text{C}$	Watts	Kg	Cm
707	434	720	4.13	770-C	0.24	(5) .02	2.5	1625-C	0.29	(5) .02
700	427	692								
675	402	598	3.62	610-C	0.28	(4) .02				
650	377	514								
625	352	440	3.30	445-C	0.27	(3) .02				
600	327	373								
575	302	315								
550	277	264	8.35	350-C	0.27	(3) .02	4.79	560-S	0.27	(3) .02
525	252	219								
500	227	180								
492	219	169								

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where  $dm$  = increase in mass

$m$  = initial mass of heat pipe

$l_c$  = length of heat pipe condenser

$l_t$  = total length of heat pipes

$T_o$  = desired operating temperature

$T$  = actual operating temperature

$T_o - T = \Delta T$  down heat pipe

From Table 3.5 and 3.6, using the lowest mass heat pipes, the increase in mass was calculated using Equation 3.2 and is tabulated in Table 3.7. Therefore, to a first approximation, one can say that the heat pipes for the CBC radiator will have a mass of 0.3 Kg each.

The performance of the mercury heat pipes is based on perfect wetting, that is, the wetting angle is zero (0). For long term stability, this may not be the case. Wetting angles from 0-60 degrees have been observed, with 30-60 degree angles, the most common. Since the capillary force is a function of the cosine of the wetting angle, the mercury heat pipes may have a reduction of capillary force of up to 50% ( $\cos 60 = .5$ ). This reduction in performance will then require a reoptimization of the heat pipes with a small increase in mass.

TABLE 3.7

INCREASE IN MASS DUE TO PERFORMANCE AT

<u>TEMPERATURE</u> (°K)	<u>POWER</u> (W)	<u>FLUID</u>	<u>MASS</u> (Kg)	<u>ΔT</u> (°C)	<u>dM</u> (Kg)	<u>NEW MASS</u> (Kg)
707	720	Hg	0.291	4.13	$4.6 \times 10^{-3}$	0.296
700	692					
675	598	Hg	0.280	3.63	$4.6 \times 10^{-3}$	0.284
650	514					
625	440	Hg	0.273	3.30	$3.9 \times 10^{-3}$	0.277
600	373					
575	315	Hg	0.273	5.81	$7.5 \times 10^{-3}$	0.280
550	264	DTA	0.280	6.49	$1 \times 10^{-2}$	0.290
525	219	DTA	0.273	4.98	$7.1 \times 10^{-3}$	0.280
500	180					
492	159	DTA	0.273	4.05	$6.2 \times 10^{-3}$	0.279

### 3.3 Advanced Heat Pipe Concept

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The groove heat pipe designs of Sections 3.1 and 3.2 optimized to an approximate mass of 0.3 Kg per heat pipe, exclusive of fins and armor. This mass is quite low and may be acceptable in the overall system. However, there are several heat pipe design concepts which may offer further reduced mass with increased performance. These include but are not limited to arterial wick heat pipes and configuration pumped heat pipes.

#### 3.3.1. Artery/Wick Heat Pipes

There is a natural division in heat pipe fluids which takes place at approximately  $600^{\circ}\text{K}$ . Above  $600^{\circ}\text{K}$ , the liquid metals are useful working fluids. Below  $600^{\circ}\text{K}$ , one generally deals with non-metallic fluids and devises structures which compensate for their inferior physical properties. The low temperature fluids, taken as a class, have relatively low latent heats of vaporization, low surface tension, and low thermal conductivity. The consequences are that for a given heat transfer rate, heat pipes using these fluids must move relatively large quantities of liquid with unusually low pressure losses, yet must maintain very thin liquid films in the heat flow path. The arterial wick structures of Figure 3.2 have been used to offset these property limitations. The artery provides the primary liquid return to the evaporator. This passage has a large hydraulic radius and provides a very low drag path. In the evaporator and condenser, a thin film of liquid is distributed circumferentially. The distribution wick is often a thin layer of screen or circumferential grooves.

The artery is removed from the evaporator and condenser heat flow paths. The thin films provided by the circumferential wick prevent the

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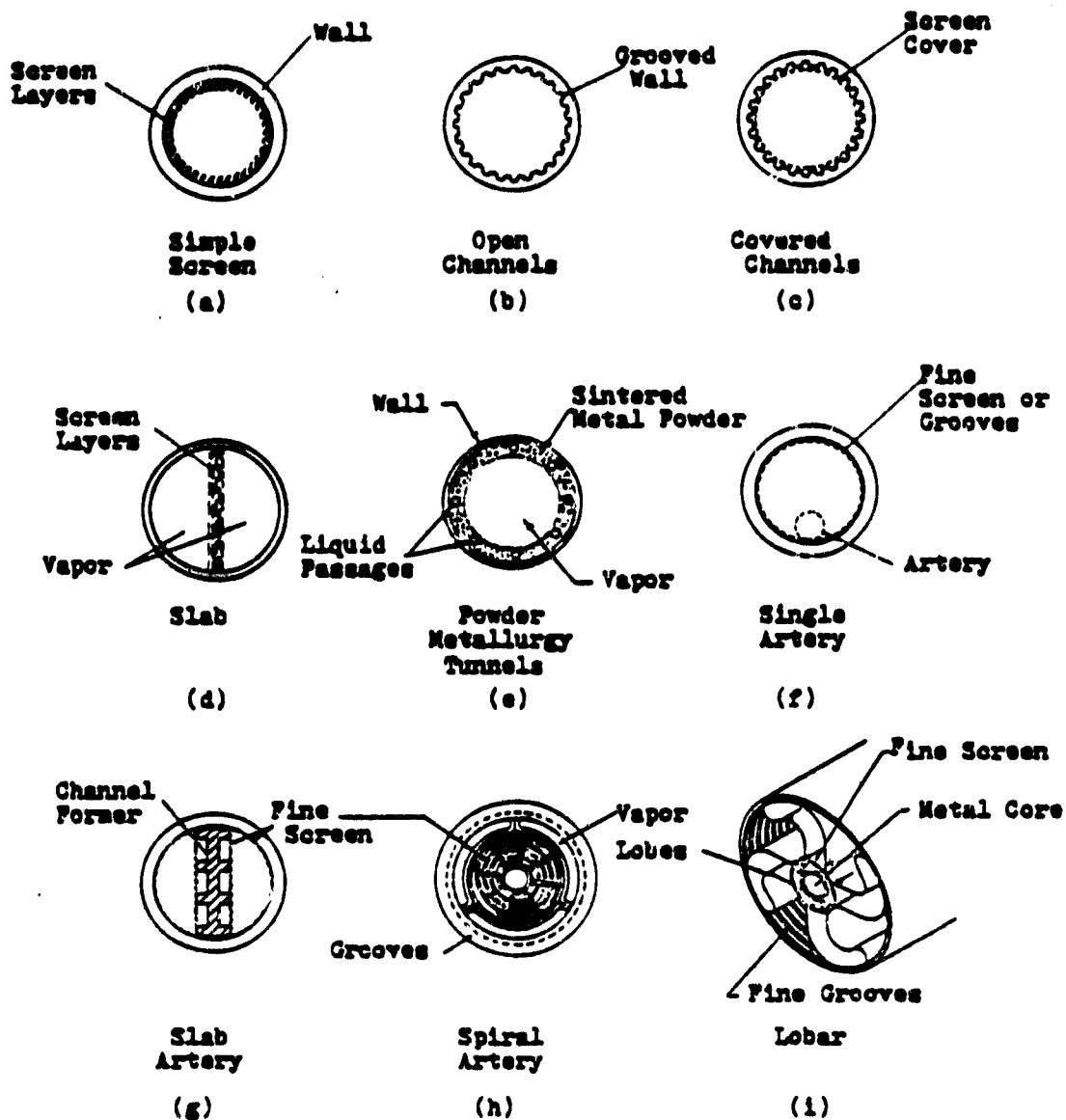


Figure 34.2 Representative Wick Geometries



development of excessive temperature gradients. Arterial wicks provide very high performance, sometimes even approaching that obtainable with liquid metals in more conventional wicks. Lengths in excess of ten meters have been reported. The primary limitations of arterial wicks lie in their difficulty of fabrication and their consequent lack of reproducible performance. The wick structures are quite difficult to form and to insert into the heat pipe vessel so as to maintain uniform close fit to the wall. There has been repeated difficulty with the priming of arteries, that is, the ability to fill an artery with fluid and keep it filled.

Two methods of priming are in use. Capillary priming, as the name implies, depends on capillary forces to maintain the fluid within the artery. The basic condition for capillary priming is that the largest single pore at the artery surface in the evaporator must provide sufficient capillary pressure to offset all counter forces including accelerations. Consequently, the evaporator ends of the arteries must be closed and there must be no single inadvertently large pore on the entire periphery of the enclosing surface. Due to the adverse effect of accelerations, capillary primed arteries can be more fractious during ground testing than in subsequent zero g operation. Yet ground testing is essential to establish the operability of the heat pipe.

If the artery is so located in the heat pipe temperature gradient that it always is the coldest spot, it will operate at a lower vapor pressure than the balance of the heat pipe. If the magnitude of the vapor pressure difference is sufficient, it will cause priming to take place. This is known as vapor pressure or Clapeyron priming. The process is highly temperature dependent. The pressure difference caused by a given temperature difference varies enormously with temperature. Thus, a heat pipe which primes reliably

and quickly at high temperature (i.e. high pressure) may fail to prime at all at low temperature. It has also been reported that vibration has caused arteries to lose their prime and that subsequent re-priming can be unreliable.

In spite of their apparent drawbacks, the performance of arterial heat pipes is sufficiently high to justify further work to improve their reliability and reproducibility. In general, arterial wicks require less total mass of wicking material, and may also require less fluid inventory than conventional heat pipes. They are, therefore, serious candidates for use in space radiators.

### 3.3.2. Wickless (Configuration Pumped) Heat Pipes

A crevice has capillary properties. Therefore, if the wall of a non-round heat pipe is formed so as to produce longitudinal crevices, these may serve the purpose of wicks. That is, the configuration of the wall provides the capillary pumping force. Several potential configuration pumped heat pipe geometries are shown in Figure 3.3. Configuration pumped heat pipes have been built (Figure 3.4) and have been shown to operate. However, there has been very little work in the field, and the mathematical prediction of performance is incomplete.

The driving pressure difference which causes liquid flow in a heat pipe is determined by the surface tension and the difference in the radius of the liquid meniscus in the condenser and evaporator. Evaporation in the heat input section tends to depress the liquid level while condensation at the heat output end tends to increase the level. Thus, during operation, the liquid level in the evaporator of a configuration pumped heat pipe recedes into the crevice, increasing the pumping pressure but decreasing

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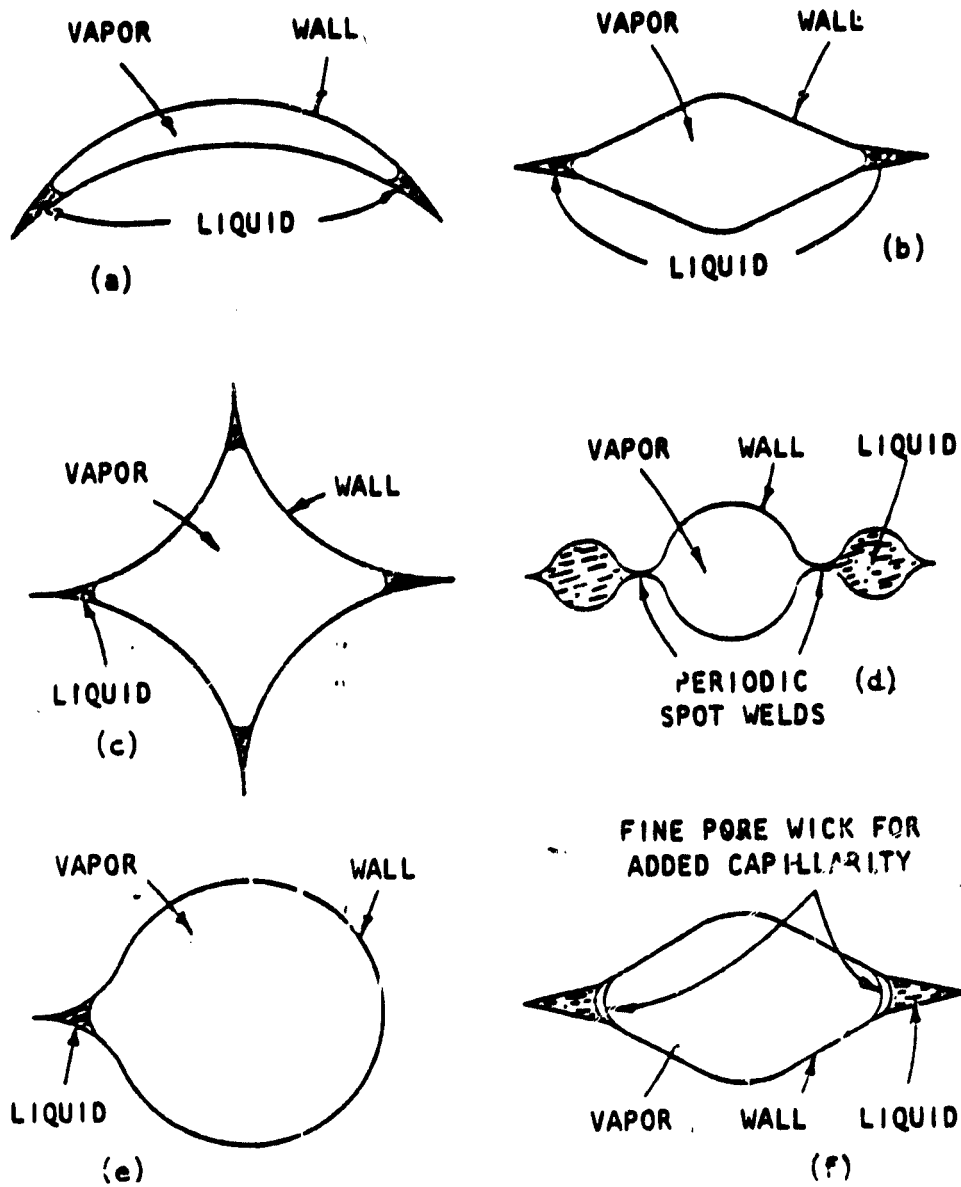


Figure 3.3 Configuration Pumped Geometries

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Photograph of a Configuration Pumped Heat Pipe  
(Courtesy of U.S. Air Force)

FIGURE 3.4

the flow area. The inverse occurs in the condenser. This makes for a delicate tradeoff of liquid fill versus power handling capability. The problem is somewhat alleviated in the configuration/artery geometry of Figures 3.3d and 3.3f.

Configuration pumped heat pipes tend by their nature to have relatively low capillary pumping forces and low liquid drag. They therefore lend themselves well to consideration as elements in low temperature space radiators where large radiating areas require long heat pipes. The liquid inventory requirement of configuration pumped heat pipes appears to be comparable to that of the arterial structures discussed previously. The complete absence of conventional wicks is a substantial mass reduction. However, the non-round shapes are relatively poor pressure vessels so that the gain in mass due to elimination of the wick may be at least partially offset by a thicker wall requirement unless fluid vapor pressures are kept relatively low. Thus the operating temperature range for a configuration pumped heat pipe of low mass may be narrower than that for other geometries.

The ability of configuration pumped heat pipes to hold their shape is a function of the creep strength of the heat pipe envelope. Thermacore<sup>2</sup> previously identified the iron alloy, A-286, which exhibits an exceptionally high creep strength, and may well serve as a containment for configuration pumped heat pipes. (A-286 has a 0.1% creep at 1100°F in 10<sup>5</sup> hours under a 38,000 psi stress load).

### 3.3.3. Hybrid Wick/Pumped Heat Pipes

Since the dissipating capacity of a space radiator declines as the fourth power of any temperature loss, there is a strong incentive to minimize

losses. One of the principal advantages of the heat pipe is the low temperature loss it incurs while moving large amounts of heat. This low  $\Delta T$  operation is characteristic of vapor heat transfer. There may, therefore, be reason to make use of vapor heat transfer even at power levels which cannot be sustained by capillary pumping alone. Alternative or hybrid pumping means are possible and deserve consideration. This may be true not only for the radiators themselves, but also for the primary loops feeding them. A practical hybrid system may use an alternative pumping means for liquid transport over appreciable distances with capillary pumping for local distribution and collection.

The heat transfer capability of a conventional heat pipe can be limited by entrainment of liquid from the walls by the high velocity, counterflowing vapor. Separation of the liquid and vapor passages will permit greater heat flow under these conditions. Figure 3.5 is a hybrid system where the liquid and vapor flow are in the same direction. Therefore, the vapor shear forces may aid rather than inhibit liquid flow.

Hybrid heat pipes are directly analogous to two-pipe steam heating systems for buildings which use condensate pumps for liquid return. The principle has been extended to liquid metals by Philips Laboratories for use in Stirling engines.

The main disadvantages of the hybrid system are the increased probability of a leak at pump seals and joints and the dependence of operation on an external power source. For maximum redundancy, there should be a pump for each heat pipe, a serious penalty in complexity for a space radiator, making the approach seem more applicable to primary loops.

It may be possible to make use of the "heat of the radiator" to pump the liquid, much the same way that a capillary pump makes use of the "heat

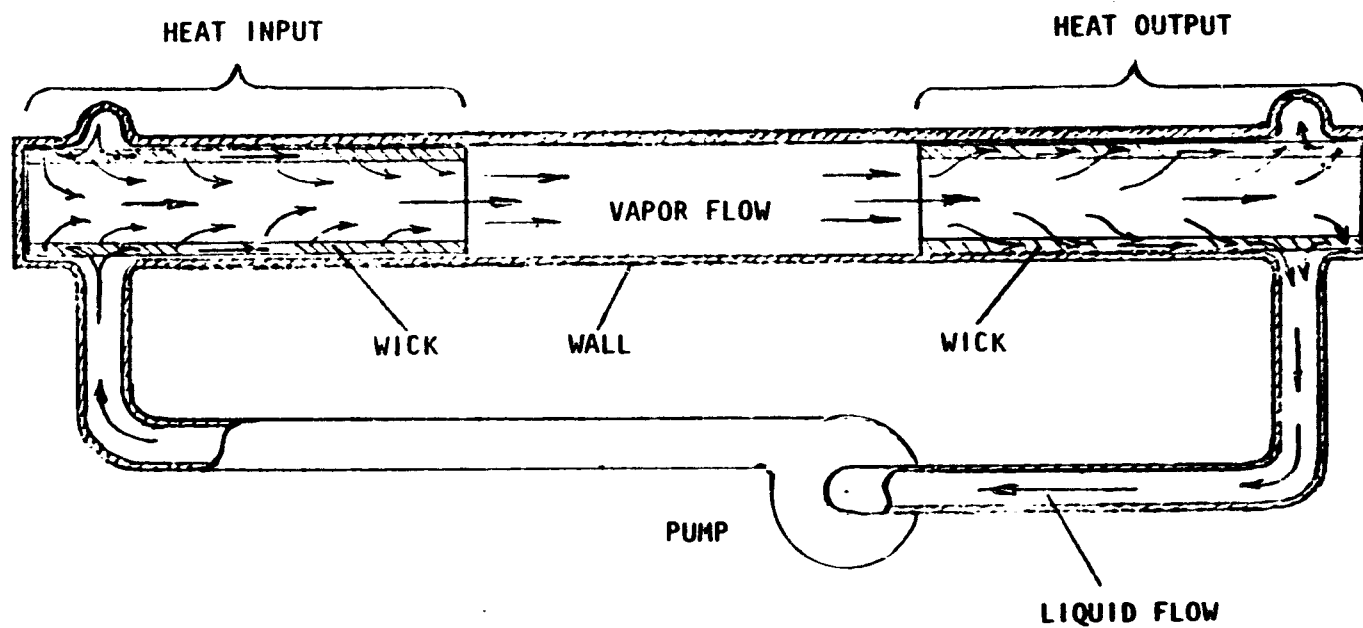


Figure 3.5 Mechanically Pumped Hybrid Heat Pipe

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of the radiator."

Thermacore has recently begun the exploration of a "liquid piston pump" as part of its internal R&D effort. This pump uses a localized high heat flux, into the fluid, to develop a vapor bubble of sufficient pressure to push the liquid forward. Backward flow is prevented by the use of a check valve. A forward spring loaded valve allows the pressure at which the pump activates to be regulated.

Initial work to date has concentrated on gravity feed liquid systems with encouraging results. The extension of this concept to two phase systems with freedom from gravity will pose challenging work but may be worth a cursory investigation.

#### 3.3.4. Other Concepts

There are numerous concepts which have been suggested as possible fluid pumping mechanisms for heat pipes and includes electro-magnetic, electrolytic, electrohydrodynamic and electrophoretic pumping. All of these are not suited for individual spacecraft radiator heat pipes. However, osmotic pumped heat pipes and artificial gravity are two possible mechanisms which are suited for spacecraft use.

If a spinning spacecraft can be so arranged that its centrifugal force will aid liquid return in heat pipes, it may be possible to eliminate pumping and depend entirely or predominantly on artificial gravity for this function. The result may be mass reduction (by wick elimination and, possibly, reduced fluid inventory) and an added degree of freedom in fluid selection (fluid need not have high surface tension).

Osmotic pressures can exceed capillary pressures by a factor of 100 to



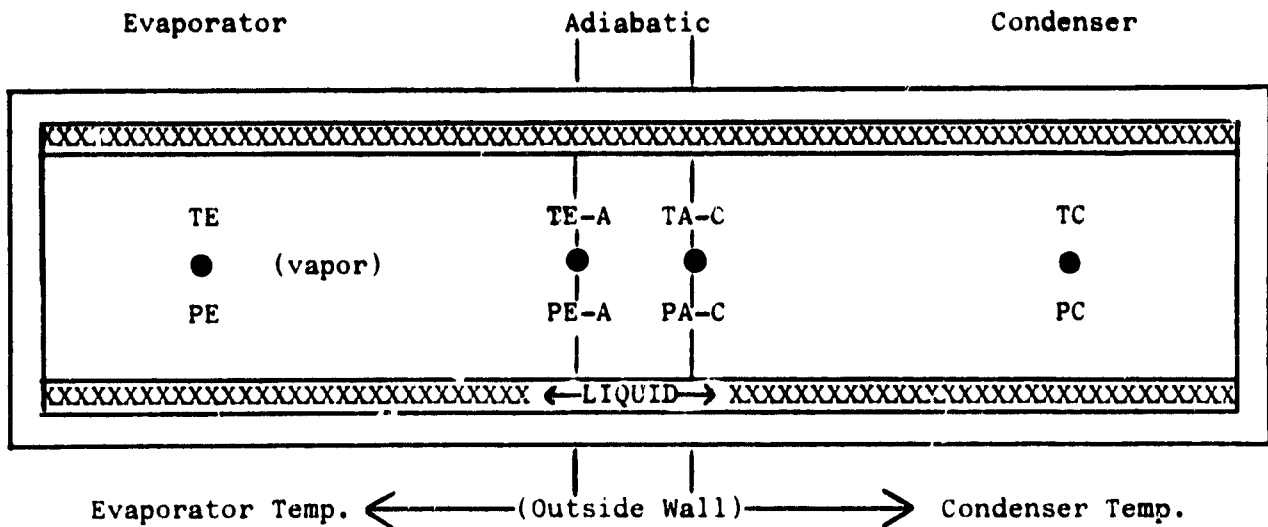
1,000. An osmotically pumped heat pipe is feasible in principle. Several designs have been proposed, but no hardware tests have been reported. The proposed designs all make use of gravity in one way or another: to keep liquid in place, to redistribute salt by natural convection, etc. It may be possible to devise a geometry which will function in gravity-free space. If so, osmotic heat pipes may avoid entirely the capillary limitations on available pumping pressure.

Flow rates through semi-permeable membranes are low; i.e., large areas are required to permit useful heat flow. There is, however, an interesting factor which may favor further consideration for low temperature space radiators. These radiators also require large areas because of the low radiant power densities. The osmotic process is such that the membrane must be located at the condenser (heat dissipating) end of the system, which is the radiating surface of a radiator. At temperatures below about  $900^{\circ}\text{K}$ , the power density from a black body radiator is less than the power density sustainable by flow of the best fluids (e.g. water) through membranes. That is, below this temperature the unit liquid flow rate through a membrane is more than sufficient to support the unit radiant heat load from a radiator of equal area, and a basic condition of successful operation has been satisfied.

The geometries considered to date are relatively massive, having two walls and a large liquid inventory. Membranes do not exist for operation above about  $400^{\circ}\text{K}$ . However, since an osmotic heat pipe would need no auxiliary power (comparable to a capillary heat pipe), it deserves further consideration.

## APPENDIX 1

This appendix has complete performance printouts of all the heat pipes tabulated in Section 3.1 and 3.2. The heat pipe program used is Thermacore's GROOVE27. Figure A.1 depicts the placement and definitions of many of the symbols in the printout.



DPVE = Pressure drop in vapor in evaporator

DPLEG = Pressure drop in liquid in evaporator grooves

DPUA = Pressure drop in vapor in adiabatic

DPLAG = Pressure drop in liquid in adiabatic grooves

DPVC = Pressure drop in vapor in condenser - (+) means drop, (-) means recovery or increase

DPLGG = Pressure drop in liquid in condenser grooves

## APPENDIX 2

This appendix develops Equation 3.2 which shows how the mass of a radiator heat pipe increases with the performance  $T$  of the heat pipe.

$T_0$  = desired heat pipe temperature

$\Delta T$  = temperature drop down heat pipe

$T = T_0 - \Delta T$ , actual heat pipe radiating temperature

$A_0$  = radiating area of heat pipe at  $T_0$

$A = A_0 + da$ , actual heat pipe radiator area required at  $T$

$Q$  = power to be radiated from heat pipe

$$\frac{da}{dt} = \frac{\text{increase in surface area}}{\text{decrease in temperature}}$$

$$\frac{da}{dt} = \frac{A - A_0}{T_0 - T} \quad \text{Eq. A.1}$$

$$\text{but} \quad A = \frac{Q}{\epsilon \sigma (T_0 - T)^4} \quad \text{and} \quad A_0 = \frac{Q}{\epsilon \sigma T_0^4}$$

therefore, with substitution into Equation A.1 and proper rearranging,

$$\frac{da}{dt} = \frac{A_0}{\Delta T} [(T_0/T)^4 - 1] \quad \text{Eq. A.2}$$

Now, since area is a function of length, we have

$$dl = l_c [(T_0/T)^4 - 1] \quad \text{Eq. A.3}$$

where  $l_c$  = condenser length, but  $\frac{dl}{l_t} = \frac{dm}{m}$  where  $l_t$  = total heat pipe length,

$m$  = mass, we obtain with substitution and rearrangement -

$$dm = \frac{ml_c}{l_t} [(T_0/T)^4 - 1] \quad \text{Eq. A.4}$$

which is Equation 3.2.

## REFERENCES

2. Garrett AiResearch, Study of a Space Nuclear Power System for a Nuclear Electric Spacecraft; JPL Contract 955008.
3. Thermacore, Inc., Letter Progress Report #3, October, 1978, Heat Pipe Heat Rejection System and Demonstration Model for the Nuclear Electric Propulsion (NEP) Spacecraft; JPL Contract 955100.

RUN CONDITIONS:

3:59 P.M. 4/ 5/79

FLUID = RUBIDIUM WALL MATL=304SS  
 EVAP TEMP = 434 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTC ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.7213 IN 176.0000 CM  
 TOTAL LENGTH 103.1800 IN 262.0000 CM

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O.D. 1.013 IN 2.5400 CM  
 WALL THICK 0.0700 IN 0.0762 CM  
 GROOVE WIDTH 0.0787 IN 0.2000 CM  
 GROOVE HEIGHT 0.0197 IN 0.0500 CM  
 LAND WIDTH 0.0344 IN 0.0875 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 720 WATTS

----- TOTAL DELTA-T = 2.86 DEG C  
 ----- TOTAL MASS = 1.749 KG

WANT PERFORMANCE DETAILS (T OR M) ??

PE	PE-A	PA-C	PC	DYNES/CM2
31291.9	30261.1	29954.9	30641.5	
TE	TE-A	TA-C	TC	DEG C
432.853	431.368	430.502	431.72	
EVAP TEMP	COND TEMP	DELTA-T		
434	431.439	2.56104		
DPC= 18214	DPC= 0	DPC+DPC= 18214	DYNES/CM2	
DPVE	DPLEB	DPVA	DPLAG	
1030	180	305	1115	
DPVC	DPLCG			
-687	739			

SONIC LIMITS: EVAP= 2197 ADD= 2487 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	2	0	142	
E R RET#	E A RET#	LIC RET#	C A RET#	C R RET#
21	3244	109	3247	5

HOT FLUID CHARGE 129.61 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 84.6019 CM3

COLD FLUID CHARGE 151.635 GRAMS  
 98.9783 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 1596.9 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.329831	.120554E-01	.608936E-02	.300293	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
1.3042	.545828	-1.21729		
CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.073367	.147544E-02	.292101E-02	.20283	

POWER OF 1775 WATTS CAUSES ----- ADD SONIC LIMIT

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 1750 WATTS

----- TOTAL DELTA-T = 7.16 DEG C  
 ----- TOTAL MASS = 1.749 KG

RUN CONDITIONS:

4:38 P.M. 3/22/78

2

FLUID = RUBIDIUM WALL MATL=304SS  
 EVAP TEMP = 377 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

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EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADB LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.0787 IN 0.2000 CM  
 GROOVE HEIGHT 0.0197 IN 0.0500 CM  
 LAND WIDTH 0.0344 IN 0.0875 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 814 WATTS

----- TOTAL DELTA-T = 6.44 DEG C  
 ----- TOTAL MASS = 1.749 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
9608.01	8292.16	7774.8	8836.82	

TE	TE-A	TA-C	TC	DEG C
378.772	367.761	364.907	370.888	

EVAP TEMP	COND TEMP	DELTA-T
377	370.887	6.44312

DPC= 19141	DPC= 0	DPC+DPC= 19141	DYNES/CM2
------------	--------	----------------	-----------

DPVE	DPLED	DPVA	DPLAG
1613	132	517	821
DPVC	DPLCG		
-1113	543		

SONIC LIMITS: EVAP= 749 ADB= 703 WATTS

WATTS=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	101	

E R RET#	E A RET#	LIG RET#	C A RET#	C R RET#
16	2443	63	2486	3

HOT FLUID CHARGE 132.019 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 96.1741 CM3

COLD FLUID CHARGE 151.638 GRAMS  
 98.9783 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 1596.9 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LCG	EVAP MESH	EVAPORATION	DEG C
.315904	.712316E-02	.353761E-02	.600536	

VAPOR (2)	VAPOR (A)	VAPOR (C)
3.01123	2.35352	-5.9802

CONDENSATION	COND MESH	COND LCG	COND WALL	DEG C
.148734	.862923E-03	.170859E-02	.181184	

POWER OF 609 WATTS CAUSES ----- ADB SONIC LIMIT

LAST NOT-LIMITED POWER CALCULATION WAS AT ----- 608 WATTS

----- TOTAL DELTA-T = 7.34 DEG C  
 ----- TOTAL MASS = 1.749 KG

RUN CONDITIONS:

4:18 P.M. 3/22/79

3

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 352 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

EVAP LENGTH 16.8291 IN 43.0000 CM  
 ADS LENGTH 16.8291 IN 43.0000 CM  
 COND LENGTH 99.2213 IN 176.0000 CM  
 TOTAL LENGTH 132.8795 IN 342.0000 CM

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O.D. 1.0000 IN 2.5400 CM  
 WALL THICK 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.0787 IN 0.2000 CM  
 GROOVE HEIGHT 0.0286 IN 0.0680 CM  
 LAND WIDTH 0.0247 IN 0.0627 CM  
 27 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT 440 WATTS

----- TOTAL DELTA-T = 3.89 DEG C  
 ----- TOTAL MASS = 1.744 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

FE	FE-A	FA-G	FG	DTHE/CM2
.546112E+07	.546109E+07	.546106E+07	.546105E+07	
TE	TE-A	TA-G	TC	DEG C
348.876	348.876	348.876	348.876	
EVAP TEMP	COND TEMP	DELTA-T		
352	348.11	3.88989		
DPC= 3160	DPC= 0	DPC+DPC= 3160	DTHE/CM2	
DPVE	DPLE	DPVA	DPLAG	
19	233	7	1339	
DPVC	DPLC			
8	986			

SONIC LIMITS: EVAP= 186826 ADS= 188977 WATTS

C/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	35	
E R REYN	E A REYN	LIG REYN	C A REYN	C R REYN
5	305	346	305	1

HOT FLUID CHARGE 114.957 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 107.637 CM3

COLD FLUID CHARGE 133.501 GRAMS  
 125.001 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 1610.42 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.635983	1.34221	.64413	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.488291E-03	.488281E-03	.244141E-03		
CONDENSATION	COND MESH	COND LAG	COND WALL	DEG C
.244557E-01	.187453	.480962	.131558	

POWER OF 350 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 345 WATTS

----- TOTAL DELTA-T = 4.79 DEG C  
 ----- TOTAL MASS = 1.744 KG

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 219 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 JTG ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1600 IN 262.0000 CM

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O.D. 1.0000 IN 2.5400 CM  
 WALL THICK 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.0747 IN 0.2000 CM  
 GROOVE HELL 0.0256 IN 0.0650 CM  
 LAND WIDTH 0.0247 IN 0.0627 CM  
 27 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 169 WATTS

----- TOTAL DELTA-T = 1.73 DEG C  
 ----- TOTAL MASS = 1.744 LB

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
402617	402492	402482	402432	

TE	TE-A	TA-C	TC	DEG C
217.618	217.615	217.612	217.609	

EVAP TEMP	COND TEMP	DELTA-T
219	217.271	1.72906

DPG= 7261	DPG= 0	JPG+DPG= 7261	DYNES/CM2

DPVE	DPLEB	DPVA	DPLAG
24	151	21	808
DPVC	DPLCG		
29	619		

SONIC LIMITS: EVAP= 15075 ADD= 17120 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	0	0	33	

E R REY#	E A REY#	LIG REY#	C A REY#	C R REY#
2	325	42	325	0

HOT FLUID CHARGE 112.977 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 106.784 CM3

COLD FLUID CHARGE 133.501 GRAMS  
 125.001 CM3

HEAT PIPE (MESH) & 2 ENDCAPS 1610.42 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.228554	.780453	.272907	.100098	

VAPOR (E)	VAPOR (A)	VAPOR (C)
.32136E-01	.292969E-02	.292969E-02

CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.244657E-01	.666662E-01	.190864	.056916	

POWER OF 715 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 710 WATTS

----- TOTAL DELTA-T = 5.37 DEG C  
 ----- TOTAL MASS = 1.744 KG



RUN CONDITIONS:

10:57 A.M. 3/28/79

FLUID - RUBIDIUM WALL MATL=304SS  
 EVAP TEMP = 434 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 VTS ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 19.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0079 IN 0.0200 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 720 WATTS

----- TOTAL DELTA-T = 2.43 DEG C  
 ----- TOTAL MASS = 1.484 KG

WANT PERFORMANCE DETAILS (Y OR N) ?Y

PE	PE-A	PA-C	PC	DYNES/CM2
31296.2	30370.4	30100.3	30715.1	

TE	TE-A	TA-C	TC	DEG C
432.861	431.242	430.762	431.849	

EVAP TEMP	COND TEMP	DELTA-T
434	431.57	2.42993

DPC= 18214	DPG= 0	DPC+DPG= 18214	DYNES/CM2
------------	--------	----------------	-----------

DPVE	DPLE	DPVA	DPLA0
926	1196	269	9336
DPVC	DPLC0		
-615	4900		

SONIC LIMITS: EVAP= 23.4 ADD= 2631 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	2	0	142	

N R REY#	E A REY#	LIG REY#	C A REY#	C R REY#
21	3161	92	3163	6

HOT FLUID CHARGE 92.1796 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 60.1694 CM3

COLD FLUID CHARGE 107.824 GRAMS  
 70.3814 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 1376.82 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L43	EVAP MESH	EVAPORATION	DEG C
.828931	.352913-02	.593373E-02	.300293	

VAPOR (E)	VAPOR (A)	VAPOR (C)
1.61914	.479736	-1.08667

CONDENSATION	COND MESH	COND L46	COND WALL	DEG C
.073367	.143844-02	.354731-03	.202311	

POWER OF 320 WATTS CAUSES ----- CAPILLARY LIMIT. DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 316 WATTS

----- TOTAL DELTA-T = 2.87 DEG C  
 ----- TOTAL MASS = 1.484 KG

RUN CONDITIONS:

8:24 A.M. 3/30/79

6

FLUID = RUBIDIUM WALL MATL=304SS  
 EVAP TEMP = 377 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HGT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0079 IN 0.0200 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 814 WATTS

----- TOTAL DELTA-T = 5.83 DEG C  
 ----- TOTAL MASS = 1.484 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
9928.78	8499.64	8061.89	9003.7	

TE	TE-A	TA-C	TC	DEG C
376.877	368.863	366.456	371.445	

EVAP TEMP	COND TEMP	DELTA-T
377	371.17	5.82983

DPC= 19139	DPG= 0	DPC+DPG= 19139	DYNES/CM2
------------	--------	----------------	-----------

DPVE	DPLEB	DPVA	DPLAG
1429	877	447	6367
DPVC	DPLCG		
-952	3599		

SONIC LIMITS: EVAP= 790 ADD= 765 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	101	

E R REYN	E A REYN	L R REYN	C A REYN	C R REYN
16	2381	54	2392	3

HOT FLUID CHARGE 93.8783 GRAM  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 61.2782 CM3

COLD FLUID CHARGE 107.824 GRAMS  
 70.3814 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 1373.82 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.615994	.203109E-02	.350565E-02	.500488	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
7.01439	2.40698	-4.9395		
CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.122278	.842339E-03	.500006E-03	.151117	

POWER OF 845 WATTS CAUSES ----- ADD SONIC LIMIT

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 640 WATTS

----- TOTAL DELTA-T = 7.13 DEG C  
 ----- TOTAL MASS = 1.484 KG

RUN CONDITIONS:

12: 2 P.M. 3/28/79

7

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 382 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTU ANG = 0.00 DEG

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EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADB LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0217 IN 0.0550 CM  
 LAND WIDTH 0.0044 IN 0.0112 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 440 WATTS

----- TOTAL DELTA-T = 9.23 DEG C  
 ----- TOTAL MASS = 1.548 LB

WANT PERFORMANCE DETAILS (Y OR N) ??

PE PE-A PA-C PC DYNES/CM2  
 .511061E+07 .511059E+07 .511058E+07 .511054E+07

TE TE-A TA-C TC DEG C  
 344.591 344.391 344.591 344.59

EVAP TEMP COND TEMP DELTA-T  
 382 342.773 9.22729

DPC= 3294 DPG= 0 DPC+DPG= 3294 DYNES/CM2

DPTE DPLE DPVA DPLAG  
 18 219 7 1430  
 DPVC DPLOC  
 7 896

SONIC LIMITS: EVAP= 180576 ADB= 181288 WATTS

Q/A'S= EVAP COND AXIAL WATTS/CM2  
 1 0 86

E R REY# E A REY# LIG REY# C A REY# C R REY#  
 5 797 291 797 1

HOT FLUID CHANGE 120.206 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHANGE 112.551 CM3

COLD FLUID CHANGE 141.404 GRAMS  
 132.401 CM3

HEAT PIPE (MESH) & 2 ENDCAPS 1406.22 GRAMS

DELTA-T VALUES:

EVAP WALL EVAP LAG EVAP MESH EVAPORATION DEG C  
 .336983 6.13104 .640463 .100098

VAPOR (A) VAPOR (B) VAPOR (C)  
 .244141E-03 .244141E-03 .244141E-03

CONDENSATION COND MESH COND LAG COND WALL DEG C  
 .244567E-01 .136647 1.50406 .132076

POWER OF 570 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 570 WATTS

----- TOTAL DELTA-T = 11.92 DEG C  
 ----- TOTAL MASS = 1.548 LB

RUN CONDITIONS:

11:50 A.M. 3/28/79

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 219 VAPOR DELTA-T = 50 DEG C  
 GRAV AMO = 0.00 WTS AMO = 0.00 DEG

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EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0197 IN 0.0500 CM  
 LAND WIDTH 0.0049 IN 0.0125 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 169 WATTS

----- TOTAL DELTA-T = 3.55 DEG C  
 ----- TOTAL MASS = 1.535 KG

WANT PERFORMANCE DETAILS (T OR H) ??

PE	PE-A	PA-C	PC	DYNES/CM2
387562	387538	387503	387479	
TE	TE-A	TA-C	TC	DEG C
216.162	216.149	216.146	216.143	
EVAP TEMP	COND TEMP	DELTA-T		
219	216.446	3.55363		
DPC= 7307	DPG= 0	DPC+DPG= 7307	DYNES/CM2	
DPVE	DPLEB	DPVA	DPLAG	
24	179	20	1200	
DPVC	DPLCB			
29	736			

SONIC LIMITS: EVAP= 14943 ADL= 16962 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	0	0	33	
E R REY#	E A REY#	LIQ REY#	C A REY#	C R REY#
2	321	36	321	0

HOT FLUID CHARGE 111.829 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 104.709 CM3

COLD FLUID CHARGE 131.942 GRAMS  
 123.841 CM3

HEAT PIPS. (MESH) & 2 ENDCAPS 1402.85 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.228554	2.24923	.269645	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.322064E-02	.292969E-02	.292969E-02		
CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.244557E-01	.658972E-01	.550474	.559992E-01	

POWER OF 569 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 553 WATTS

----- TOTAL DELTA-T = 11.48 DEG C  
 ----- TOTAL MASS = 1.535 KG

# RUN CONDITIONS:

4: 7 P.M. 4/ 5/79

FLUID = RUBIDIUM WALL MATL=304SS  
 EVAP TEMP = 434 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 WTC ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADB LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

ORIGINAL PAGE IS  
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O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0300 IN 0.0762 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0079 IN 0.0200 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 720 WATTS

----- TOTAL DELTA-T = 2.43 DEG C  
 ----- TOTAL MASS = 1.484 KG

## WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
31296.8	30370.4	30100.3	30715.1	

TE	TE-A	TA-C	TC	DEG C
432.861	431.242	430.762	431.849	

EVAP TEMP	COND TEMP	DELTA-T
434	431.57	2.42993

DPC= 18214	DPC= 0	DPC+DTC= 18214	DYNES/CM2
------------	--------	----------------	-----------

DPVE	DPLEB	DPVA	DPLAG
926	1196	269	9336
DPVC	DPLCQ		
-615	4900		

SONIC LIMITS: EVAP= 2314 ADB= 2631 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	2	0	142	

A R RET#	E A RET#	LIG RET#	C A RET#	C R RET#
21	3161	92	3163	5

HOT FLUID CHARGE 92.1796 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 60.1634 CM3

COLD FLUID CHARGE 107.824 GRAMS  
 70.3814 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 1375.82 GRAMS

## DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.329831	.382612E-02	.593373E-02	.300293	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
1.61314	.479736	-1.08667		
CONDENSATION	COND MESH	COND LAG	COND WALL	DEG C
.073367	.143842E-02	.85473E-03	.202811	

POWER OF 820 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION 433 AT ----- 315 WATTS

----- TOTAL DELTA-T = 2.37 DEG C  
 ----- TOTAL MASS = 1.484 KG

RUN CONDITIONS:

11: 3 A.M. 3/28/79

10

FLUID - RUBIDIUM WALL MATL-304SS  
 EVAP TEMP = 377 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 WTC ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADB LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0100 IN 0.0254 CM  
 GROOVE WIDTH 0.1063 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0129 IN 0.0328 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 514 WATTS

----- TOTAL DELTA-T = 4.56 DEG C  
 ----- TOTAL MASS = 0.691 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
10019.5	8848.97	8495.41	9239.51	
TE	TE-A	TA-C	TC	DEG C
376.293	370.666	368.84	372.611	
EVAP TEMP	COND TEMP	DELTA-T		
377	372.438	4.56226		
DPC= 19133	DPG= 0	DPC+DPG= 19133	DYNES/CM2	
DPVE	DPLED	DPVA	DPLAG	
1170	876	363	6860	
DPVC	DPLCG			
-745	3697			

SONIC LIMITS: EVAP= 867 ADB= 870 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	101	
E R REYN	E A REYN	LIC REYN	C A REYN	C R REYN
16	2281	58	2290	3

HOT FLUID CHARGE 95.35 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 62.5863 CM3

COLD FLUID CHARGE 110.102 GRAMS  
 71.8683 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 580.457 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.201125	.207981E-02	.336022E-02	.500489	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
5.82695	1.32544	-3.77051		
CONDENSATION	COND MESH	COND LAG	COND WALL	DEG C
.122278	.810205E-03	.801449E-03	.492989E-01	

POWER OF 710 WATTS CAUSES ----- ADB SONIC LIMIT

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 705 WATTS

----- TOTAL DELTA-T = 6.12 DEG C  
 ----- TOTAL MASS = 0.691 KG

CONDITIONS:

4:14 P.M. 4/ 8/79

ID = DOWNHELM A WALL MATL=304SS  
P TEMP = 352 VAPOR DELTA-T = 50 DEG C  
V ANG = 0.00 WTG ANG = 0.00 DEG

P LENGTH 16.9291 IN 43.0000 CM  
D LENGTH 69.2913 IN 176.0000 CM  
AL LENGTH 103.1500 IN 262.0000 CM

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L THICK 1.0000 IN 2.5400 CM  
L THICKS 0.0100 IN 0.0254 CM  
OVE WIDTH 0.1083 IN 0.2750 CM  
OVE HEIGHT 0.0217 IN 0.0550 CM  
D WIDTH 0.0094 IN 0.0240 CM  
GROOVES (CLOSED) COVERED WITH 200 MESH

LIMIT ENCOUNTERED AT ----- 440 WATTS

----- TOTAL DELTA-T = 8.72 DEG C  
--- TOTAL MASS = 0.777 KG

T PERFORMANCE DETAILS (Y OR N) ??

	PE-A	PA-C	PC	DYNES/CM2
33874E+07	.533872E+07	.533872E+07	.533869E+07	
	TE-A	TA-C	TC	DEG C
7.403	347.402	347.402	347.402	
P TEMP	COND TEMP	DELTA-T		
2	346.275	5.72485		
= 3206	DPG= 0	DPC+DPG= 3206	DYNES/CM2	
E	DPLEE	DPVA	DPLAG	
	218	6	1426	
C	DPLCG			
	894			

IC LIMITS: EVAP= 170527 ADB= 208679 WATTS

'S=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	86	
REY#	E A REY#	LIC REY#	C A REY#	C R REY#
	764	287	764	1

FLUID CHARGE 123.66 GRAMS  
TEMP. VOLUME OF HOT FLUID CHARGE 116.787 CM3

FLUID CHARGE 142.992 GRAMS  
133.888 CM3

P! (MESH) & 2 ENDCAPS 634.002 GRAMS

TA-T VALUES:

WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
5328	3.70969	.611878	.100098	
OR (E)	VAPOR (A)	VAPOR (C)		
5281E-03	.244141E-03	0		
CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
4537E-01	.149862	.909027	.430151E-01	

OF 560 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

NON-LIMITED POWER CALCULATION WAS AT ----- 555 WATTS

RUN CONDITIONS:

8:49 A.M. 3/30/79

12

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 219 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTG ANG = 0.00 DEG

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EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADS LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 1.0000 IN 2.5400 CM  
 WALL THICKS 0.0100 IN 0.0254 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0217 IN 0.0550 CM  
 LAND WIDTH 0.0094 IN 0.0240 CM  
 25 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 160 WATTS

----- TOTAL DELTA-T = 2.49 DEG C  
 ----- TOTAL MASS = 0.777 LG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
396240	396219	396194	396170	
TE	TE-A	TA-C	TC	DEG C
217.008	217.006	217.003	217.001	
EVAP TEMP	COND TEMP	DELTA-T		
219	216.514	2.48647		
DPC= 7280	DPG= 0	DPC+DPG= 7280	DYNES/CM2	
DPVE	DPLE	DPVA	DPLAG	
20	141	17	926	
DPVC	DPLCG			
24	580			

SONIC LIMITS: EVAP= 16500 ADE= 18735 WATTS

3/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	0	0	33	
E R REY#	E A REY#	LIQ REY#	C A REY#	C R REY#
2	308	36	308	0

HOT FLUID CHARGE 121.123 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 113.411 CM3

COLD FLUID CHARGE 142.992 GRAMS  
 133.888 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 634.002 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.746241E-01	1.85778	.259088	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.277712E-02	.244141E-02	.244141E-02		
CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.244557E-01	.633176E-01	.781132	.182887E-01	

POWER OF 715 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 710 WATTS

----- TOTAL DELTA-T = 10.06 DEG C  
 ----- TOTAL MASS = 0.777 LG



RUN CONDITIONS:

12:00 P.M. 3/27/79

FLUID = MERCURY WALL MATL=304SS  
 EVAP TEMP = 434 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

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 OF POOR QUALITY

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 0.3750 IN 0.9524 CM  
 WALL THKSS 0.0039 IN 0.0100 CM  
 GROOVE WIDTH 0.0787 IN 0.2000 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0592 IN 0.1504 CM  
 8 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 720 WATTS

----- TOTAL DELTA-T = 2.69 DEG C  
 ----- TOTAL MASS = 0.449 EG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
.386688E+07	.386703E+07	.386601E+07	.38667E+07	
TE	TE-A	TA-C	TC	DEG C
431.868	431.839	431.82	431.833	
EVAP TEMP	COND TEMP	DELTA-T		
434	431.311	2.68872		
DPC= 114106	DPG= 0	DPC+DPG= 114106	DYNES/CM2	
DPVE	DPLEB	DPVA	DPLAG	
1547	6854	1007	51200	
DPVC	DPLCG			
-682	28057			

SONIC LIMITS: EVAP= 20067 ADE= 24069 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	3	1	1010	
E R RET#	E A RET#	LIC RET#	C A RET#	C R RET#
13	5074	330	5074	3

HOT FLUID CHARGE 281.891 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 20.8099 CM3

COLD FLUID CHARGE 290.48 GRAMS  
 21.444 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 158.896 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LUG	EVAP MESH	EVAPORATION	DEG C
.284282	.732296	1.01444	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.290527E-01	.192871E-01	-.129396E-01		
CONDENSATION	COND MESH	COND LUG	COLD WALL	DEG C
.244557E-01	.248109	.179159	.695814E-01	

POWER OF 935 WATTS CAUSES ----- CAPILLARY LIMIT, CPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 930 WATTS

----- TOTAL DELTA-T = 3.44 DEG C  
 ----- TOTAL MASS = 0.449 EG

RUN CONDITIONS:

10:41 A.M.

3/30/79

FLUID = MERCURY  
 EVAP TEMP = 352  
 GRAV ANG = 0.00  
 WALL MATL=304SS  
 VAPOR DELTA-T = 50 DEG C  
 WTD ANG = 0.00 DEG

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EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADS LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 0.3750 IN 0.9525 CM  
 WALL THICKS 0.0039 IN 0.0100 CM  
 GROOVE WIDTH 0.0787 IN 0.2000 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0692 IN 0.1506 CM  
 8 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 440 WATTS

----- TOTAL DELTA-T = 1.98 DEG C  
 ----- TOTAL MASS = 0.449 EG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYSES/CM2
.116902E+07	.116769E+07	.116638E+07	.116601E+07	
TE	TE-A	TA-C	TC	DEG C
350.888	350.437	350.335	350.308	
EVAP TEMP	COND TEMP	DELTA-T		
352	350.023	1.97729		
DPC= 120072	DFG= 0	DPC+DFG= 120072	DYSES/CM2	
DPVE	DPLEB	DPVA	DPLAG	
1926	4347	1313	32473	
DPVC	DPLCB			
-434	17796			

SONIC LIMITS: EVAP= 6206 ADD= 7012 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	3	0	617	
E R REY#	E A REY#	LIG REY#	C A REY#	C R REY#
9	3479	187	3480	2

HOT FLUID CHARGE 282.835 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 20.8796 CM3

COLD FLUID CHARGE 290.5 GRAMS  
 21.4454 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 158.621 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.184184	.472935	.63448	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.150635	.102539	-.336914E-01		
CONDENSATION	COND MESH	COND LAG	COND WALL	DEG C
.244357E-01	.150059	.115683	.450586E-01	

POWER OF 305 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 300 WATTS

----- TOTAL DELTA-T = 3.63 DEG C  
 ----- TOTAL MASS = 0.449 EG

RUN CONDITIONS:

1:13 P.M. 3/27/79

16

FLUID = MERCURY WALL MATL=304SS  
 EVAP TEMP = 277 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 VTC ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 89.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.3750 IN 0.9524 CM  
 WALL THICKS 0.0039 IN 0.0100 CM  
 GROOVE WIDTH 0.0787 IN 0.2000 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0592 IN 0.1504 CM  
 8 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 264 WATTS

----- TOTAL DELTA-T = 2.08 DEG C  
 ----- TOTAL MASS = 0.449 LG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
258510	258701	253728	253630	
TE	TE-A	TA-C	TC	DEG C
276.066	276.542	276.168	276.15	
EVAP TEMP	COND TEMP	DELTA-T		
277	274.921	2.0791		
DPC= 127000	DPG= 0	DPC+DPG= 127000	DYNES/CM2	
DPVE	DPLEG	DPVA	DPLAG	
2808	2705	1971	20211	
DPVC	DPLCG			
98	11077			

SONIC LIMITS: EVAP= 1500 ADD= 1693 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	2	0	370	
E R REY#	E A REY#	LIG REY#	G A REY#	C R REY#
6	2436	105	2441	1

HOT FLUID CHARGE 284.166 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 20.9779 CM3

COLD FLUID CHARGE 290.43 GRAMS  
 21.444 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 168.396 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L2G	EVAP MESH	EVAPORATION	DEG C
.116944	.300587	.416003	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.523926	.373779	.135647E-01		
CONDENSATION	COND MESH	COND L2G	COND WALL	DEG C
.244657E-01	.101791	.735424E-01	.286171E-01	

POWER OF 310 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPG

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 303 WATTS

----- TOTAL DELTA-T = 3.33 DEG C  
 ----- TOTAL MASS = 0.449 LG

RUN CONDITIONS:

2:57 P.M. 3/27/79

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 352 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

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EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 0.5000 IN 1.2700 CM  
 WALL THICKS 0.0050 IN 0.0127 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0298 IN 0.0750 CM  
 LAND WIDTH 0.0046 IN 0.0116 CM  
 12 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 140 WATTS

----- TOTAL DELTA-T = 15.04 DEG C  
 ----- TOTAL MASS = 0.306 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PE-C	PC	DYTES/CM2
.474816E+07	.474784E+07	.474760E+07	.474762E+07	
TE	TE-A	TE-C	TC	DEG C
339.936	339.931	339.929	339.928	
EVAP TEMP	COND TEMP	DELTA-T		
352	336.989	15.0408		
DPC= 3439	DPC= 0	DPC+DPC= 3439	DYTES/CM2	
DPVE	DPLE	DPVA	DPLA0	
316	220	146	1330	
DPVC	DPLC0			
68	903			

SONIC LIMITS: EVAP= 32864 ADD= 39214 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	2	0	347	
E R REY#	E A REY#	LIG REY#	C A REY#	C R REY#
5	1656	552	1656	1

HOT FLUID CHARGE 66.0229 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 61.8192 CM3

COLD FLUID CHARGE 96.3628 GRAMS  
 30.8641 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 219.701 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LUG	EVAP MESH	EVAPORATION	DEG C
.175328	10.452	1.33681	.100098	
VAPOR (B)	VAPOR (A)	VAPOR (C)		
.430483E-02	.213727E-02	.732422E-03		
COND MESH	COND LUG	COND WALL		
.244675E-01	.326322	.667392	.433123E-01	DEG C

POWER OF 515 WATTS CAUSES ----- CAPILLARY LIMIT, DEL > DPC

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 510 WATTS

----- TOTAL DELTA-T = 17.42 DEG C  
 ----- TOTAL MASS = 0.306 KG

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 327 VAPOR DELTA-T = 60 DEG C  
 PRAY ANG = 0.00 W% ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 60.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.8000 IN 1.2700 CM  
 WALL THICKS 0.0080 IN 0.0127 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0286 IN 0.0660 CM  
 LAND WIDTH 0.0066 IN 0.0168 CM  
 12 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 375 WATTS

----- TOTAL DELTA-T = 10.73 DEG C  
 ----- TOTAL MASS = 0.294 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
.332091E+07	.332063E+07	.332047E+07	.332037E+07	
TE	TE-A	TA-C	TC	DEG C
318.394	318.386	318.385	318.384	
EVAP TEMP	COND TEMP	DELTA-T		
327	316.27	10.7302		
DPC= 4112	DPC= 0	DPC+DTC= 4112	DYNES/CM2	
DPVE	DPLX	DPVA	DPLAG	
283	274	150	1722	
DPYC	DPLC			
103	1125			

SONIC LIMITS: EVAP= 24880 ADD= 29626 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	2	0	294	
E R REY#	E A REY#	LIG REY#	C A REY#	C R REY#
4	1372	405	1372	1

HOT FLUID CHARGE 60.2462 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 56.4103 CM3

COLD FLUID CHARGE 77.4415 GRAMS  
 72.5108 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 216.768 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.131407	7.22392	1.15017	.100068	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.337109E-02	.292969E-02	.135013E-02		
CONDENSATION	COND MESH	COND LAG	COND WALL	DEG C
.244577E-01	.275341	1.77523	.372296E-01	

POWER OF 121 WATTS CAUSES ----- CAPILLARY LIMIT. DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 420 WATTS

----- TOTAL DELTA-T = 12.06 DEG C  
 ----- TOTAL MASS = 0.294 KG

NUM CONDITIONS:

2:31 P.M. 3/27/79

17

FLUID = DOWTHERM A WALL MATL=30433  
 EVAP TEMP = 302 VAPOR DELTA-T = 80 DEG C  
 GPAT ANG = 0.00 VTS ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 175.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.8000 IN 1.2700 CM  
 WALL THICK 0.0060 IN 0.0127 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0236 IN 0.0300 CM  
 LAND WIDTH 0.0076 IN 0.0194 CM  
 12 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 318 WATTS

----- TOTAL DELTA-T = 8.36 DEG C  
 ----- TOTAL MASS = 0.288 EG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE .222571E+07	PE-A .222542E+07	PA-C .222521E+07	PG .222506E+07	DYNES/CM2
TE 296.287	TE-A 296.279	TA-C 296.274	TC 296.271	DEG C
EVAP TEMP 302	COND TEMP 293.623	DELTA-T 8.37695		
DPC= 4834	DPC= 0	DPC+DPA= 4834	DYNES/CM2	
DPVS 280	DPLM 308	DPVA 168	DPLAG 1988	
DPVC 160	DPLG 1262			

SONIC LIMITS: EVAP= 17710 ADD= 20631 WATTS

Q/A'S=	EVAP 1	COND 0	AXIAL 248	WATTS/CM2
E R REY# 3	E A REY# 1161	LIG REY# 233	C A REY# 1161	C R REY# 0

HOT FLUID CHARGE 57.6718 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 63.9968 CM3

COLD FLUID CHARGE 72.9808 GRAMS  
 68.3341 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 214.816 GRAMS

DELTA-T VALUES:

EVAP WALL .130299	EVAP LOG 8.52068	EVAP MESH .962189	EVAPORATION .100998	DEG C
VAPOR (E) .756836E-02	VAPOR (A) .488281E-02	VAPOR (C) .366211E-02		
CONDENSATION .214537E-01	COND MESH .235245	COND LOG 1.35508	COND WALL .032033	DEG C

POWER OF 375 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 370 WATTS

----- TOTAL DELTA-T = 9.82 DEG C  
 ----- TOTAL MASS = 0.288 EG

# RUN CONDITIONS:

2:24 P.M. 3/27/79

19

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 277 VAPOR DELTA-T = 80 DEG C  
 GRAV AMG = 0.00 WTC AMG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADS LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.8000 IN 1.2700 CM  
 WALL THICKS 0.0050 IN 0.0127 CM  
 GROOVE WIDTH 0.1083 IN 0.2760 CM  
 GROOVE HEIGHT 0.0217 IN 0.0560 CM  
 LAND WIDTH 0.0087 IN 0.0220 CM  
 12 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 264 WATTS

----- TOTAL DELTA-T = 6.40 DEG C  
 ----- TOTAL MASS = 0.291 KG

## WANT PERFORMANCE DETAILS (Y OR N) ??

FE	FE-A	FA-C	FC	DTHEM/CM2
.139913E+07	.139989E+07	.139907E+07	.139946E+07	
FE	TE-A	TA-C	TC	DEG C
271.61	271.790	271.701	271.784	
EVAP TEMP	COND TEMP	DELTA-T		
277	270.511	6.48877		
DPC= 5568	DPC= 0	DPC+DPC= 5568	DTHEM/CM2	
DPVE	DPLEG	DPVA	DPLAG	
289	351	193	2298	
DPVC	DPLCG			
208	1438			

SONIC LIMITS. EVAP= 11918 ADS= 13775 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
1	0	208		
E R REYS	E A REYS	LIG REYS	C A REYS	C R REYS
3	965	194	965	0

HOT FLUID CHARGE 55.1152 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 51.606 CM3

COLD FLUID CHARGE 68.5201 GRAMS  
 51.1575 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 212.637 GRAMS

## DELTA-T VALUES:

EVAP WALL	EVAP L.G	EVAP MESH	EVAPORATION	DEG C
.111323	1.16533	.31347	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.109863E-01	.786830E-02	.702422E-02		
CONDENSATION	COND MESH	COND L.G	COND WALL	DEG C
.244567E-01	.19857	1.02143	.273353E-01	

POWER OF 310 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NOT-LIMITED POWER CALCULATION WAS AT ----- 305 WATTS

----- TOTAL DELTA-T = 7.48 DEG C  
 ----- TOTAL MASS = 0.291 KG

FLUID = DOWTHERM A WALL MATL=304SS  
 EVAP TEMP = 282 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.8000 IN 1.2700 CM  
 WALL THICKS 0.0080 IN 0.0127 CM  
 GROOVE WIDTH 0.1093 IN 0.2750 CM  
 GROOVE HEIGHT 0.0197 IN 0.0500 CM  
 LAND WIDTH 0.0097 IN 0.0247 CM  
 12 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 219 WATTS

----- TOTAL DELTA-T = 4.98 DEG C  
 ----- TOTAL MASS = 0.274 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PG	DYSES/CM2
837793	837486	837235	836952	
TE	TE-A	TA-C	TO	DEG C
248.035	248.017	248.004	247.998	
EVAP TEMP	COND TEMP	DELTA-T		
282	247.015	4.98198		
DPG= 6311	DPG= 0	DPG+DPG= 6311	DYSES/CM2	
DPVZ	DPLEB	DPVA	DPLAG	
307	413	231	2761	
DPVC	DPLCS			
283	1693			

SONIC LIMITS: EVAP= 7617 ADD= 8733 WATTS

1/1'S=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	172	

E R REY#	E A REY#	LIO REY#	C A REY#	C R REY#
2	806	131	806	0

HOT FLUID CHARGE 82.8281 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 49.1836 CM3

COLD FLUID CHARGE 64.0596 GRAMS  
 39.9808 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 209.931 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.941753E-01	3.09005	.031114	.100099	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.172119E-01	.131836E-01	.148926E-01		
CONDENSATION	COND MESH	COND LAG	COND WALL	DEG C
.244557E-01	.166508	.78715	.230995E-01	

POWER OF 245 WATTS CAUSES ----- CAPILLARY LIMIT, DFL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 240 WATTS

----- TOTAL DELTA-T = 5.45 DEG C  
 ----- TOTAL MASS = 0.274 KG



RUN CONDITIONS:

2: 4 P.M.

3/27/79

21

FLUID = DOWTHERM A WALL MATL=3041S  
 EVAP TEMP = 219 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 WTC ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADL LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.8000 IN 1.2700 CM  
 WALL THICKS 0.0080 IN 0.0127 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0197 IN 0.0500 CM  
 LAND WIDTH 0.0097 IN 0.0247 CM  
 12 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 169 WATTS

----- TOTAL DELTA-T = 4.05 DEG C  
 ----- TOTAL MASS = 0.274 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
384494	384095	383751	383296	

TE	TE-A	TA-C	TC	DEG C
215.845	215.805	215.771	215.724	

EVAP TEMP	COND TEMP	DELTA-T
219	214.952	4.04629

DPC= 7317	DPC= 0	DPC+DPC= 7317	DYNES/CM2
-----------	--------	---------------	-----------

DPVE	DPLES	DPVA	DPLAG
388	375	334	2805
DPVC	DPLCG		
465	1536		

SONIC LIMITS: EVAP= 3713 ADB= 4207 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	0	0	133	

E R REYN	E A REYN	LIC REYN	C A REYN	C R REYN
2	643	76	643	0

HOT FLUID CHARGE 53.9219 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 50.4837 CM3

COLD FLUID CHARGE 64.0596 GRAMS  
 59.9908 CM3

HEAT PIPE. (MESH) & 2 ENDCAPS 209.931 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.746241E-01	2.44127	.53893	.100093	

VAPOR (E)	VAPOR (A)	VAPOR (C)
.398712E-01	.344238E-01	.468309E-01

CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.241537E-01	.131746	.397897	.018292	

POWER OF 220 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 215 WATTS

----- TOTAL DELTA-T = .12 DEG C  
 ----- TOTAL MASS = 0.274 KG

RUN CONDITIONS:

9:36 A.M.

3/30/70

FLUID = MERCURY  
 EVAP TEMP = 434  
 GRAV ANG = 0.00  
 WALL MATL=304SS  
 VAPOR DELTA-T = 30 DEG C  
 WTD ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 80.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.2500 IN 0.6350 CM  
 WALL THICKS 0.0025 IN 0.0064 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0358 IN 0.0909 CM  
 8 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 720 WATTS

----- TOTAL DELTA-T = 4.13 DEG C  
 ----- TOTAL MASS = 0.290 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE .381500E+07	PE-A .360703E+07	PA-C .379924E+07	PC .380308E+07	DYTES/CM2
TE 430.878	TE-A 430.712	TA-C 430.564	TC 430.637	DEG C
EVAP TEMP 434	COND TEMP 429.872	DELTA-T 4.12042		
DPC= 114169	DPG= 0	DPC+DPG= 114169	DYTES/CM2	
DPVE 8650	DPLEG 7271	DPVA 7766	DPLAG 56688	
DPVC -3844	DPLCG 29763			

SONIC LIMITS: EVAP= 8444 ADD= 10060 WATTS

Q/A'S=	EVAP 8	COND 2	AXIAL 2273	WATTS/CM2
S R REY# 13	S A REY# 7785	LIC REY# 393	C A REY# 7787	C R REY# 3

HOT FLUID CHARGE 206.068 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 15.2117 CM3

COLD FLUID CHARGE 213.035 GRAMS  
 15.7268 CM3

HEAT PIPE (MESH) & 2 ENDCAPS 76.483 GRAMS

DELTA-T VALUES:

EVAP WALL .270618	EVAP LAG 1.19874	EVAP MESH 1.53447	EVAPORATION .100098	DEG C
VAPOR (B) .163318	VAPOR (A) .147517	VAPOR (C) -.72508E-01		
CONDENSATION .244857E-01	COND MESH .380819	COND LAG .293439	COND WALL .863033E-01	DEG C

POWER OF 775 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DAV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- TWO WATTS

----- TOTAL DELTA-T = 4.12 DEG C  
 ----- TOTAL MASS = 0.290 KG

ROOM CONDITIONS:

9:44 A.M. 3/30/79

23

FLUID = MERCURY WALL MATL=304SS  
 EVAP TEMP = 402 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 WTU ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADD LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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D. 0.2500 IN 0.6350 CM  
 WALL THICKS 0.0028 IN 0.0064 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0718 IN 0.1823 CM  
 4 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 508 WATTS

----- TOTAL DELTA-T = 3.62 DEG C  
 ----- TOTAL MASS = 0.231 EG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE 39459E+07	PE-A .238535E+07	PA-C .237686E+07	PC .238027E+07	DYNES/CM2
TE 399.401	TE-A 399.153	TA-C 398.924	TC 399.016	DEG C
EVAP TEMP 402	COND TEMP 398.379	DELTA-T 3.62061		
DPC= 116291	DPG= 0	DPC+DPG= 116291	DYNES/CM2	
DPVE 9242	DPLEG 7655	DPVA 8462	DPLAG 59687	
DPVC -3408	DPLCG 31338			

SONIC LIMITS: EVAP= 5424 ADD= 6418 WATTS

Q/A'S=	EVAP 6	COND 1	AXIAL 1888	WATTS/CM2
E R REY# 11	E A REY# 6726	LIQ REY# 397	C A REY# 6729	C R REY# 2

HOT FLUID CHARGE 187.61 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 13.8499 CM3

COLD FLUID CHARGE 193.515 GRAMS  
 14.2858 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 87.8669 GRAMS

DELTA-T VALUES:

EVAP WALL 229854	EVAP L&G .950525	EVAP MESH 1.31773	EVAPORATION .100038	DEG C
VAPOR (2) .248291	VAPOR (A) .223004	VAPOR (C) -.917969E-01		
CONDENSATION .244557E-01	COND MESH .322469	COND L&G .232703	COLD WALL .562394E-01	DEG C

POWER OF 615 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 610 WATTS

----- TOTAL DELTA-T = 3.69 DEG C  
 ----- TOTAL MASS = 0.231 EG

RUN CONDITIONS:

9:52 A.M.

3/30/79

24

FLUID - MERCURY WALL MATL=304SS  
 EVAP TEMP = 382 VAPOR DELTA-T = 50 DEG C  
 GRAV ANG = 0.00 VTS ANG = 0.00 DEG

EVAP LENGTH 16.9291 IN 43.0000 CM  
 ADB LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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O.D. 0.2500 IN 0.6350 CM  
 WALL THICKS 0.0025 IN 0.0064 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.1318 IN 0.3348 CM  
 3 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 440 WATTS

----- TOTAL DELTA-T = 3.30 DEG C  
 ----- TOTAL MASS = 0.273 EG

WANT PERFORMANCE DETAILS (Y OR N) Y

PE	PE-A	PE-S	PS	DYNES/CM2
.116282E+07	.116188E+07	.116172E+07	.116428E+07	

TE	TE-A	TE-S	TS	DEG C
380.04	348.883	348.046	348.179	

EVAP TEMP	COND TEMP	DELTA-T
382	348.699	3.30103

DPC= 120118	DPC= 0	DPC+DPC= 120118	DYNES/CM2
-------------	--------	-----------------	-----------

DPVE	DPLE	DPVA	DPLAG
10774	7684	10121	89918
DPVC	DPLC		
-2506	31464		

SONIC LIMITS: EVAP= 2622 ADB= 2921 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	8	1	1389	

E R RET#	E A RET#	LIC RET#	C A RET#	C R RET#
9	5337	372	5345	2

HOT FLUID CHARGE 169.292 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 12.4976 CM3

COLD FLUID CHARGE 173.995 GRAMS  
 12.8448 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 99.2507 GRAMS

DELTA-T VALUES:

EVAP WALL	EVAP LAG	EVAP MESH	EVAPORATION	DEG C
.175328	.630505	1.00319	.100098	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
.487031	.537354	-.133057		
CONDENSATION	COLD MESH	COND LAG	COND WALL	DEG C
.244557E-01	.215535	.166603	.429384E-01	

POWER OF 450 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 445 WATTS

----- TOTAL DELTA-T = 3.34 DEG C  
 ----- TOTAL MASS = 0.273 EG

# IN CONDITIONS:

9:38 A.M. 4/ 5/79

LIQUID = MERCURY  
 VAP TEMP = 302  
 RAV ANG = 0.00  
 WALL MATL=304SS  
 VAPOR DELTA-T = 50 DEG C  
 WTC ANG = 0.00 DEG

VAP LENGTH 16.9291 IN 43.0000 CM  
 DB LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

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OF POOR QUALITY

O.D. 0.2500 IN 0.6350 CM  
 WALL THICKNESS 0.0025 IN 0.0064 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.1318 IN 0.3348 CM  
 3 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 316 WATTS

----- TOTAL DELTA-T = 4.51 DEG C  
 ----- TOTAL MASS = 0.273 KG

## WANT PERFORMANCE DETAILS (T OR M) ??

PE	PE-A	PA-C	PC	DYNES/CM2
445637	432020	418926	419952	

TE	TE-A	TA-C	TC	DEG C
300.518	299.124	297.747	297.356	

EVAP TEMP	COND TEMP	DELTA-T
302	297.493	4.50708

DPC= 124573	DPG= 0	DPC+DPG= 124573	DYNES/CM2
-------------	--------	-----------------	-----------

DPVE	DPLEG	DPVA	DPLAG
13616	5634	13092	43953
DPVC	DPLCG		
-1027	23093		

SOME LIMITS: EVAP= 1062 ADB= 1146 WATTS

W/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	3	0	994	

E R REY#	E A REY#	LIQ REY#	C A REY#	C R REY#
?	4203	254	4232	1

HOT FLUID CHARGE 169.39 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 12.5417 CM3

COLD FLUID CHARGE 173.995 GRAMS  
 12.8443 CM3

HEAT PIPE. (MESH) & 2 ENDCAPS 99.2507 GRAMS

## DELTA-T VALUES:

EVAP WALL	EVAP L&G	EVAP MESH	EVAPORATION	DEG C
.130209	.505764	.745609	.100093	

VAPOR (E)	VAPOR (A)	VAPOR (C)
1.39453	1.37646	-1.09375

CONDENSATION	COND MESH	COND L&G	COND WALL	DEG C
.244557 E-01	.182729	.123971	.319433 E-01	

POWER OF 390 WATTS CAUSES ----- CAPILLARY LIMIT, DPL > SPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 386 WATTS

----- TOTAL DELTA-T = 5.51 DEG C  
 ----- TOTAL MASS = 0.273 KG

NUM CONDITIONS:

12:29 P.M. 3/30/79

FLUID = MERCURY WALL MATL=304SS  
 EVAP TEMP = 277 VAPOR DELTA-T = 80 DEG C  
 GRAV ACC = 0.00 VTS ACC = 0.00 DMG

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 OF POOR QUALITY

EVAP LENGTH 13.9291 IN 43.0000 CM  
 ADS LENGTH 16.9291 IN 43.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

O.D. 0.2600 IN 0.6350 CM  
 WALL THICKS 0.0025 IN 0.0064 CM  
 GROOVE WIDTH 0.1003 IN 0.2780 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.1318 IN 0.3348 CM  
 3 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 264 WATTS

----- TOTAL DELTA-T = 8.36 DEG C  
 ----- TOTAL MASS = 0.273 KG

WANT PERFORMANCE DETAILS (Y OR N) ??

PE 256642	PE-A 240893	PA-C 224894	PC 224898	DYNES/CM2
TE 275.719	TE-A 272.845	TA-C 268.961	TC 268.962	DEG C
EVAP TEMP 277	COND TEMP 268.646	DELTA-T 8.36382		
DPC= 127035	DPO= 0	DPC+DPO= 127035	DYNES/CM2	
DPVE 16048	DPLEB 4781	DPVA 15896	DPLAG 37334	
DPVC -4	DPLCB 19636			

SONIC LIMITS: EVAP= 634 ADS= 630 WATTS

Q/A'S=	EVAP 3	COND 0	AXIAL 833	WATTS/CM2
--------	-----------	-----------	--------------	-----------

E R REY# 6	E A REY# 3737	LIC REY# 208	C A REY# 3801	C R REY# 1
---------------	------------------	-----------------	------------------	---------------

HOT FLUID CHARGE 170.187 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 12.5636 CM3

COLD FLUID CHARGE 173.996 GRAMS  
 12.8448 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 99.2607 GRAMS

DELTA-T VALUES:

EVAP WALL .111323	EVAP LAG .432317	EVAP MESH .837575	EVAPORATION .100098	DEG C
VAPOR (E) 3.17383	VAPOR (A) 3.58325	VAPOR (C) -.732422E-03		
CONDENSATION .244557E-01	COND MESH .130789	COND LAG .106314	COND WALL .273751E-01	DEG C

POWER OF 350 WATTS CAUSES ----- CAPILLARY LIMIT. DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 345 WATTS

----- TOTAL DELTA-T = 12.83 DEG C  
 ----- TOTAL MASS = 0.273 KG

RUN CONDITIONS:

12:46 P.M. 3/30/72

2"

FLUID = MERCURY WALL MATL=304SS  
 EVAP TEMP = 434 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

EVAP LENGTH 33.8583 IN 86.0000 CM  
 ADS LENGTH 0.0000 IN 0.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1500 IN 262.0000 CM

OF POOR QUALITY

O.D. 0.2504 IN 0.6360 CM  
 WALL THICKNESS 0.0025 IN 0.0064 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.0360 IN 0.0915 CM  
 5 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 720 WATTS

----- TOTAL DELTA-T = 2.80 DEG C  
 ----- TOTAL MASS = 0.290 KG

VANT PERFORMANCE DETAILS (Y OR N) ??

PE PE-A PA-C PC DYNMS/CM2  
 .389659E+07 .389724E+07 .389721E+07 .389694E+07

TE TE-A TA-C TC DEG C  
 432.39 432.216 432.216 432.286

EVAP TEMP COND TEMP DELTA-T  
 434 431.499 2.50244

DPC= 114073 DRG= 0 DPC+DRG= 114073 DYNMS/CM2

DPVE DPLEB DPVA DPLAG  
 9361 14532 0 0  
 DPVC DPLCG  
 -3725 29'42

SONIC LIMITS: EVAP= 8646 ADB= 10334 WATTS

Q/A'S= EVAP COND AXIAL WATTS/CM2  
 4 2 2266

E R REY# E A REY# LIQ REY# C A REY# C R REY#  
 8 7768 394 7769 3

HOT FLUID CHARGE 206.234 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 15.2247 CM3

COLD FLUID CHARGE 213.233 GRAMS  
 15.7414 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 76.7047 GRAMS

DELTA-T VALUES:

EVAP WALL EVAP L&G EVAP MESH EVAPORATION DEG C  
 .125096 .598018 .775944 .100098

VAPOR (D) VAPOR (A) VAPOR (C)  
 .174072 .483281E-03 -.693359E-01

CONDENSATION COND MESH COND L&G COND WALL DEG C  
 .489113E-01 .379483 .292541 .661243E-01

POWER OF 1650 WATTS CAUSES ----- CAPILLARY LIMIT. DPL > DPV

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 1625 WATTS

----- TOTAL DELTA-T = 3.47 DEG C  
 ----- TOTAL MASS = 0.290 KG

# RUN CONDITIONS:

12:54 P.M. 3/30/79

FLUID = MERCURY WALL MATL=304SS  
 EVAP TEMP = 277 VAPOR DELTA-T = 80 DEG C  
 GRAV ANG = 0.00 WTS ANG = 0.00 DEG

EVAP LENGTH 33.8883 IN 86.0000 CM  
 ADS LENGTH 0.0000 IN 0.0000 CM  
 COND LENGTH 69.2913 IN 176.0000 CM  
 TOTAL LENGTH 103.1800 IN 262.0000 CM

ORIGINAL

O.D. 0.2800 IN 0.6350 CM  
 WALL THICKS 0.0028 IN 0.0064 CM  
 GROOVE WIDTH 0.1083 IN 0.2750 CM  
 GROOVE HEIGHT 0.0079 IN 0.0200 CM  
 LAND WIDTH 0.1318 IN 0.3348 CM  
 3 GROOVES (CLOSED) COVERED WITH 200 MESH

NO LIMIT ENCOUNTERED AT ----- 264 WATTS

----- TOTAL DELTA-T = 4.79 DEG C  
 ----- TOTAL MASS = 0.273 LB

## WANT PERFORMANCE DETAILS (Y OR N) ??

PE	PE-A	PA-C	PC	DYNES/CM2
256631	240821	240817	240617	
TE	TE-A	TA-C	TC	DEG C
276.309	272.592	272.592	272.85	
EVAP TEMP	COND TEMP	DELTA-T		
277	272.21	4.78965		
DPC= 126975	DPG= 0	DPC+DPG= 126975	DYNES/CM2	
DPVE	DPLEB	DPVA	DPLAG	
19010	9659	0	0	
DPVC	DPLCG			
199	19601			

SONIC LIMITS: EVAP= 641 ADS= 632 WATTS

Q/A'S=	EVAP	COND	AXIAL	WATTS/CM2
	1	0	833	
E R REY#	E A REY#	LIQ REY#	C A REY#	C R REY#
3	3731	208	3766	1

HOT FLUID CHARGE 170.18 GRAMS  
 ROOM TEMP. VOLUME OF HOT FLUID CHARGE 12.5631 CM3

COLD FLUID CHARGE 173.995 GRAMS  
 12.8448 CM3

HEAT PIPE, (MESH) & 2 ENDCAPS 99.2507 GRAMS

## DELTA-T VALUES:

EVAP WALL	EVAP LWC	EVAP MESH	EVAPORATION	DEG C
.556614E-01	.216152	.318761	.100038	
VAPOR (E)	VAPOR (A)	VAPOR (C)		
3.7108	.732422E-03	.419922E-01		
CONDENSATION	COND MESH	COND LWC	COND WALL	DEG C
.499113E-01	.156322	.106007	.272991E-01	

POWER OF 500 WATTS CIRCLES ----- ADS SONIC LIMIT

LAST NON-LIMITED POWER CALCULATION WAS AT ----- 495 WATTS

----- TOTAL DELTA-T = 0.64 DEG C  
 ----- TOTAL MESH = 0.273 LB





**AIRESEARCH MANUFACTURING COMPANY OF ARIZONA**  
A DIVISION OF THE GARRETT CORPORATION  
PHOENIX, ARIZONA

**APPENDIX B**

**HEAT PIPE COOLED NUCLEAR  
REACTOR DESIGN INFORMATION  
FROM  
LOS ALAMOS SCIENTIFIC LABORATORY**

**(34 Pages)**

HEAT-PIPE COOLED NUCLEAR  
REACTOR DESIGN INFORMATION  
FROM  
LOS ALAMOS SCIENTIFIC LABORATORY

This appendix contains the parametric information concerning heat-pipe-cooled reactor weights and sizes for use in the NASA Brayton power plant studies which was supplied by LASL. Data on gas cooled reactors was also furnished but not included herewith since such reactors received only very cursory attention in this study. The mass summary in Figure B-1 indicates that 90%UC-10%ZrC fueled reactors are lighter but more limited in temperature than 60%UO<sub>2</sub>-40%Mo fueled reactors. Gas cooled reactors tend to be heavier below 1 MW<sub>t</sub> for 90%UC-10%ZrC and 4 MW<sub>t</sub> for 60%UO<sub>2</sub>-40%Mo.

For heat-pipe reactors, an allowance of 100°K was made for the temperature drop from the reactor heat pipes to the Brayton loop gas. This resulted in analyzing heat pipe reactors 100 degrees higher than the desired turbine inlet temperature. The turbine inlet temperatures AiResearch specified were 1150, 1325, 1500 and 1650°K. The accompanying tabulations provides information at various operating levels. It should be noted that the reactor mass includes one meter of heat pipes beyond the core for use in the heat exchanger but does not include the remainder of the heat exchanger. This mass can be adjusted as needed using the heat pipe mass/unit length values.

Both 90%UC-10%ZrC and 60%UO<sub>2</sub>-40%Mo fueled reactors with lifetimes of 10 years at full power were investigated. For the 90%UC-10%ZrC, excessive fuel swelling becomes a problem at 1425°K above 1 MW<sub>t</sub>. For lower temperatures and power levels, reactor sizes are limited by criticality and heat transfer considerations. For the region where excess swelling limitations govern, the power density in the fuel must be reduced. A number of means were examined including changing the void fraction in the fuel, reducing the <sup>235</sup>U enrichment, adjusting the heat pipe size and modifying the cladding matrix. Adjusting the void fraction will lead to the lowest weight core but at present it is only

an engineering estimate as to how much void can be accepted in a given design. It was concluded that the uncertainties and difficulties in design would not warrant designing a 90%UC-10%ZrC core if the power level and temperature exceeded 2 MW<sub>t</sub> and 1425°K since the weight was approaching that of 60%UO<sub>2</sub>-40%Mo at these conditions and would probably exceed it by 4 MW<sub>t</sub>.

The 60%UO-40%Mo reactor is criticality and heat transfer limited for the 1425, 1600, and 1750°K outlet temperature cases except that above 2 MW<sub>t</sub> for 1750°K it becomes fuel-swelling limited. Based on our current best information on fuel swelling, a 4 MW<sub>t</sub> reactor operating at 1750°K will have about 14 percent dense fuel swelling.

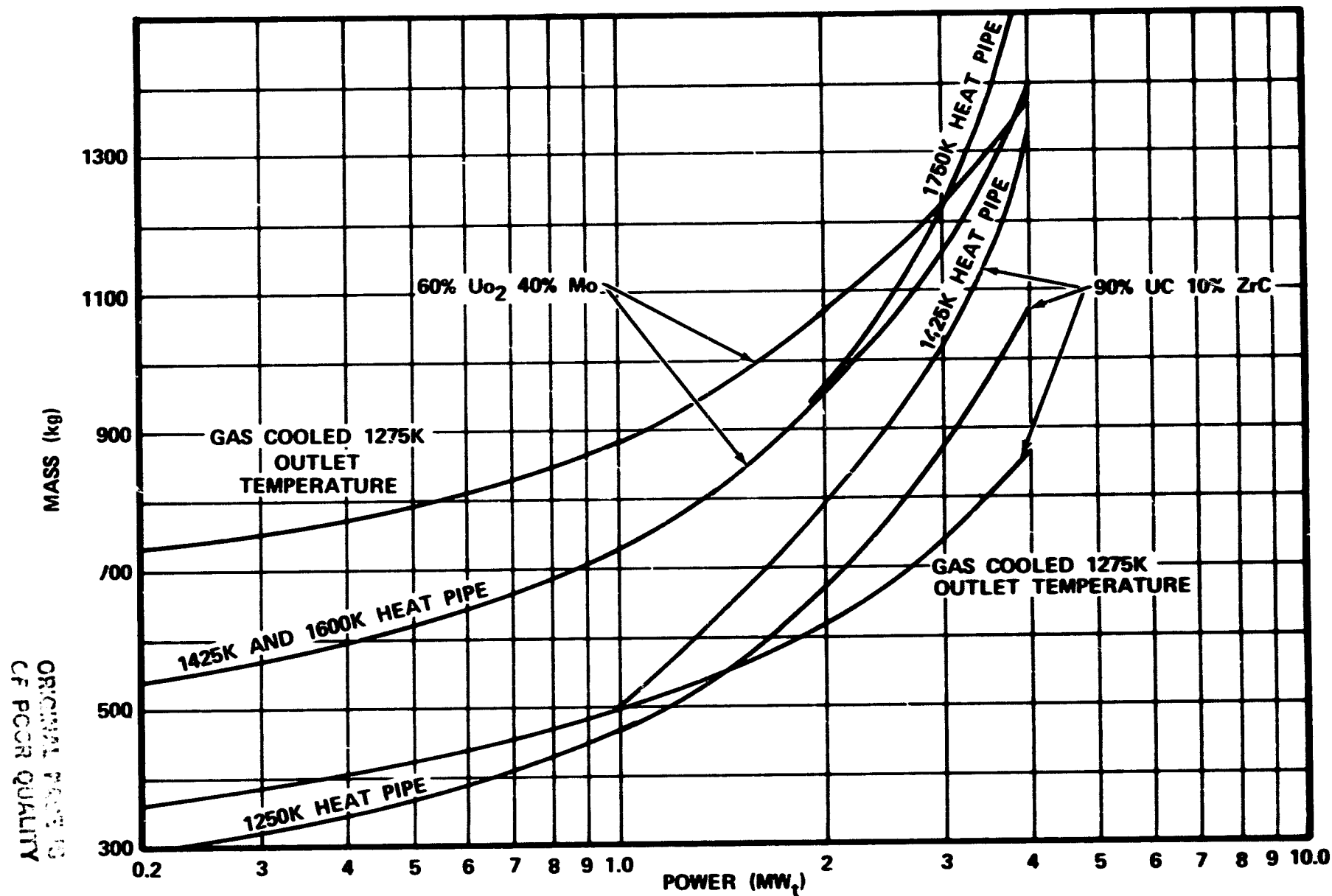


Figure B-1. Reactor Mass Summary.

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PROG NO. 2 5-17-78 TIME NEW INPUT: PR#1. NHP#2 ...STOP  
STOP

0.200 (PM) REACTOR POWER:MW (VCONF) (1.2/UC+UC2) CORE =UC  
1250. (TMP) HEAT PIPE TEMP:DEG K (NHP) (1.2/SE+SE2) REFLECTOR =SE2  
3650. (TIME) LIFETIME:DAY (NHP) (1.2/3/NS+MO+H) HEAT PIPE =NO  
1.00 (ELD) CORE L/D RATIO (VAPOR) (1.2/LI+NA) VAPOR =NA  
10.0 (DAXL) AXIAL HT FLUX:MW/CM2 (IOPTN) (1.2) OPTION =2  
200. (DTNAX) MAX FUEL DELTA T:DEG K  
1.00 (NPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN: 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING: DCONF(M) NHP(M) UNFT FBETA NPIPE ..STOP  
NPIPE#4 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PRAXIS SHIN DXMIN CORCAP ENDGAP  
0.150 0.006 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. VCD=0. PRAXIS=2. ETC ...STOP

UC=0.012 STOP

ELD INDEX = 3

UCF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.595 0.943 +.

DCH = 0. 0. 0.192 0.135 0.110 0.095 0.085 0.078 0.072 0.067

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO...84 PIPES

BETA =0.1500 UCF =0.2743 DC =0.7257 DCH =0.1000 DC =0.2237

REACTIVITY CHANGES: DELTA K

BURN = 0.00718 EXP = 0.01512 SAFE = 0.02000 TOTAL = 0.04230

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0.0500 0.8642 0.0858 0.0343 0.0515

HEXAGONAL CORNER CORRECTION FACTOR =1.0082

NUMBER OF HEAT PIPES = 35.3550

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY:DEGREE KELVIN

MAXIMUM FUEL DELTA T = 84.2

AVG DELTA T ACROSS HEAT PIPE WALL = 4.7

AVERAGE FUEL TEMPERATURE =1282.8

MAXIMUM FUEL TEMPERATURE =1341.2

BURN FRACTION OF U235 =0.0120

FSSION DENSITY (FISSIONS/CM\*\*3) = 2.492e+20

FUEL SHELLING:VOLUME % = 0.92

" " ,dense fuel% = 1.08

REACTOR DIMENSIONS: METERS

FUEL ELEMENT DIMENSIONS: MM

0.2237	CORE DIAMETER	23.11	WIDTH ACROSS HEX PLATE
0.2237	CORE HEIGHT	24.26	EQUIV. FUEL ELEMENT DIA
0.4537	REACTOR DIAMETER	23.65	EQUIV. FUEL REGION O.D.
0.4337	REACTOR HEIGHT	7.11	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	5.51	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	23.80	VAPOR AREA: MM**2
1.3287	TOTAL HEAT PIPE LENGTH		
1.4337	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

82.6 FUEL: U235 MASS = 70.2

148.7 REFLECTOR

18.1 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 13.60

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

19.8 SUPPORT STRUCTURE (7% OF REACTOR WT)

302.1 TOTAL REACTOR + HEAT PIPES

26.64 MW/M\*\*3:AVG POWR IN FUELSPACE 2.38 MW/POWER PER HEAT PIPE

100.03 MW/M\*\*2:INTPIPE AXIAL HT FLUX 0.615 MW/M\*\*2:INTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

DU

\*\*\*\*\*

PROG NO. 3 5-17-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
PR=0.4 STOP

0.400 (PR) REACTOR POWER: MW (KCORE) (1.2/UC+UC2) CORE =UC  
1250. (THR) HEAT PIPE TEMP: DEG K (KREF) (1.2/RE+REO) REFLECTOR =REO  
3650. (TIME) LIFETIME: DAYS (KHA) (1.2/3/NE+NO+H) HEAT PIPE =NO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR =NA  
10.0 (DAXL) AXIAL HT FLUX: KW/CM2 (IOPTN) (1.2) OPTION =2  
200. (DTFMAY) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(M) KREF(M) UNFT FBETA NPIPE ..STOP  
NPIPE=84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC UCD ALFA PKAVG BMIN DXMIN CORCOR ENDCAP  
0.150 0.012 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0, PKAVG=2, ETC ...STOP

STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.585 0.943 \*

DCH = 0. 0. 0.272 0.191 0.156 0.135 0.120 0.110 0.102 0.095

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1500 UNF =0.3295 UF =0.6705 DX =0.1000 DC =0.2381

REACTIVITY CHANGES: DELTA K

BURN = 0.01288 EXP = 0.01512 SAFE = 0.02000 TOTAL = 0.04800

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0.0500 0.7984 0.1516 0.0606 0.0910

HEXAGONAL CORNER CORRECTION FACTOR =1.0258

NUMBER OF HEAT PIPES = 48.6978

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 115.9

AVG DELTA T ACROSS HEAT PIPE WALL = 8.8

AVERAGE FUEL TEMPERATURE =1297.5

MAXIMUM FUEL TEMPERATURE =1379.2

BURN FRACTION OF U235 =0.0215

FISSION DENSITY (FISSIONS/CM\*\*3) = 4.474e+20

FUEL SWELLING: VOLUME % = 1.86

" " dense fuel % = 2.19

REACTOR DIMENSIONS: METERS

FUEL ELEMENT DIMENSIONS: MM

0.2381	CORE DIAMETER	24.59	WIDTH ACROSS HEX FLATS
0.2381	CORE HEIGHT	25.82	EQUIV. FUEL ELEMENT DIA
0.4681	REACTOR DIAMETER	25.17	EQUIV. FUEL REGION O.D.
0.4481	REACTOR HEIGHT	10.05	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	7.79	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	47.64	VAPOR AREA: MM**2
1.3431	TOTAL HEAT PIPE LENGTH		
1.4481	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

92.0 FUEL: U235 MASS = 78.2

162.1 REFLECTOR

36.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 27.21

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

22.7 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----

346.3 TOTAL REACTOR + HEAT PIPES

47.83 MW/CM\*\*3: AVG POWR IN FUELSPACE 4.76 KW: POWER PER HEAT PIPE

99.96 MW/CM\*\*2: HTPIPE AXIAL HT FLUX 0.817 MW/CM\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

50

\*\*\*\*\*  
 PROB NO. 4 5-17-78 TYPE NEW INPUT: PR#1. KHP#2 ...STOP  
 PR#0.7 STOP

0.700 (PR) REACTOR POWER: MW (KCOMP) (1.2/UC+UC2) CORE #UC  
 1250. (THR) HEAT PIPE TEMP: DEG K (KREF) (1.2/RE+REO) REFLECTOR #REO  
 3450. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NE+NO+W) HEAT PIPE #NO  
 1.00 (FLD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR #NA  
 10.0 (GAXL) AXIAL HT FLUX: KW/CM2 (IORTN) (1.2) OPTION #2  
 200. (DTMAX) MAX FUEL DELTA T: DEG K  
 1.00 (HALI) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE PT DESIGN; 2-SPECIFIED DESIGN  
 TYPE IN ANY OF FOLLOWING: DCORE(M) XREF(M) UNFT FBETA NPIPE ...STOP  
 NPIPE=120 STOP

120 (NPIPE) NO. OF HEAT PIPES

BETA UC UCD ALFA PKAVG BMIN DXMIN CORGAP ENDGAP  
 0.150 0.012 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP

UC=0.008 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.535 0.943 +.

DCH = 0. 3.484 0.354 0.251 0.205 0.177 0.159 0.145 0.134 0.126

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE 50 OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

50

BETA = 0.1500 UNF = 0.3905 UF = 0.6095 DX = 0.1000 DC = 0.2568

REACTIVITY CHANGES: DELTA K

BURN = 0.01977 EXP = 0.01512 SAFE = 0.02000 TOTAL = 0.05489

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
 0.0500 0.7229 0.2271 0.0908 0.1363

HEXAGONAL CORNER CORRECTION FACTOR = 1.0588

NUMBER OF HEAT PIPES = 60.6776

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 120

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 101.1

AVG DELTA T ACROSS HEAT PIPE WALL = 10.0

AVERAGE FUEL TEMPERATURE = 1293.7

MAXIMUM FUEL TEMPERATURE = 1366.2

BURN FRACTION OF U235 = 0.0329

FISSION DENSITY (FISSIONS/CM\*\*3) = 6.365E+20

FUEL SWELLING VOLUME % = 2.77

" " , dense fuel % = 3.26

REACTOR DIMENSIONS: METERS

0.2568 CORE DIAMETER 22.24 WIDTH ACROSS HEX FLATS  
 0.2568 CORE HEIGHT 23.35 EQUIV. FUEL ELEMENT DIA  
 0.4868 REACTOR DIAMETER 22.76 EQUIV. FUEL REGION O.D.  
 0.4668 REACTOR HEIGHT 11.13 HEAT PIPE O.D.  
 0.1000 REFLECTOR THICKNESS 8.62 VAPOR DIAMETER  
 1.0000 PIPE LENGTH OUTSIDE REACTOR 58.34 VAPOR AREA: MM\*\*2  
 1.3618 TOTAL HEAT PIPE LENGTH  
 1.4668 OVERALL REACTOR+HEAT PIPE LENGTH

REACTOR WEIGHTS: KILOGRAMS

104.9 FUEL: U235 MASS = 89.2

180.4 REFLECTOR

64.6 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 47.61

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

26.8 SUPPORT STRUCTURE (7% OF REACTOR WT)

409.9 TOTAL REACTOR + HEAT PIPES

73.39 MW/M\*\*3: AVG POWR IN FUELSpace 5.83 KW: POWER PER HEAT PIPE

99.98 MW/M\*\*2: HPIPE AXIAL HT FLUX 0.839 MW/M\*\*2: HPIPE RAD HTFLX

\*\*\*\*\*  
 TYPE 50 OR STOP

GO

\*\*\*\*\*

PROG NO. 5 5-17-78 TYPE NEW INPUT: PR#1, KPR#2 ...STOP  
PR#1. STOP

1.000 (PR) REACTOR POWER MW (KCORE) (1.2/UC)UC#2 CORE #UC  
1250. (THR) HEAT PIPE TEMP: DEG K (KREF) (1.2/PE)PE#2 REFLECTOR #PE#2  
3650. (TIME) LIFETIME: DAYS (KHE) (1.2/3/NB)NB#2 HEAT PIPE #MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI)LI#2 VAPOR #NA  
10.0 (GAYL) AXIAL HT FLUX: KW/CM2 (IOPTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN, 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(M) KREF(M) UNFT FBETA NPIPE ...STOP  
NPIPE=162 STOP

162 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG BMIN DXMIN CORGAP ENDGAP  
0.150 0.008 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP, OR NEW CONSTANTS IE. UC=0, PKAVG#2, ETC ...STOP

UC=0.006 STOP

SLD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.585 0.943 +.

DCM = 0. 2.606 0.419 0.298 0.244 0.212 0.189 0.173 0.160 0.150

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO

BETA = 0.1500 VNF = 0.4383 VF = 0.5617 DX = 0.1000 DC = 0.2737

REACTIVITY CHANGE: DELTA K

BURN = 0.02533 EXP = 0.01512 SAFE = 0.02000 TOTAL = 0.06045

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0.0500 0.6648 0.2852 0.1141 0.1711

HEXAGONAL CORNER CORRECTION FACTOR = 1.0943

NUMBER OF HEAT PIPES = 68.6911

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 162

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 84.8

AVG DELTA T ACROSS HEAT PIPE WALL = 9.9

AVERAGE FUEL TEMPERATURE = 1288.2

MAXIMUM FUEL TEMPERATURE = 1349.7

BURN FRACTION OF U235 = 0.0422

FISSION DENSITY (FISSIONS/CM\*\*3) = 8.796E+20

FUEL SWELLING: VOLUME % = 3.39

" " , dense fuel % = 3.99

REACTOR DIMENSIONS: METERS

FUEL ELEMENT DIMENSIONS: MM

0.2737 CORE DIAMETER 20.42 WIDTH ACROSS HEX FLATS

0.2737 CORE HEIGHT 21.44 EQUIV. FUEL ELEMENT DIA

0.5037 REACTOR DIAMETER 20.89 EQUIV. FUEL REGION O.D.

0.4837 REACTOR HEIGHT 11.45 HEAT PIPE O.D.

0.1000 REFLECTOR THICKNESS 8.87 VAPOR DIAMETER

1.0000 PIPE LENGTH OUTSIDE REACTOR 61.75 VAPOR AREA: MM\*\*2

✓ 1.3787 TOTAL HEAT PIPE LENGTH

1.4837 OVERALL REACTOR+HEAT PIPE LENGTH

REACTOR HEIGHTS: KILOGRAMS

117.0 FUEL: U235 MASS = 99.4

197.8 REFLECTOR

93.8 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 68.02

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

30.9 SUPPORT STRUCTURE (7% OF REACTOR WT)

472.5 TOTAL REACTOR + HEAT PIPES

94.02 MW/M\*\*3: AVG POWR IN FUELSPACE 6.17 KW/POWER PER HEAT PIPE

99.97 MW/M\*\*2: HTRIPE AXIAL HT FLUX 0.810 MW/M\*\*2: HTRIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP



80

PROG NO. 6 5-17-78 TYPE NEW INPUT: PR#1. KHP#2 ...STOP  
PR#2. STOP

2.000 (PR) REACTOR POWER:MW (KCORE) (1,2/UC:UC2) CORE #UC  
1250. (THP) HEAT PIPE TEMP:DEG K (KREF) (1,2/RE:RE2) REFLECTOR #RE2  
3650. (TIME) LIFETIME:DAY (KHP) (1,2/3/NB:NO:W) HEAT PIPE #NO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI:NA) VAPOR #NA  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1,2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING : DCORE(N) KREF(N) UNFT FBETA NPIPE ..STOP  
NPIPE=210 STOP

210 (NPIPE) NO. OF HEAT PIPES  
BETA UC VCD ALFA PKAVG BMIN DXMIN CORCOR ENDCOR  
0.150 0.006 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP  
UC=0.005 STOP

SLD INDEX = 3  
UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000  
DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.585 0.943 +.  
DCH = 0. 3.228 0.590 0.421 0.345 0.299 0.268 0.244 0.226 0.212

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

80  
BETA =0.1500 UNF =0.5467 VF =0.4533 DX =0.1000 DC =0.3210

REACTIVITY CHANGES: DELTA K  
BURN = 0.03890 EXP = 0.01512 SAFE = 0.02000 TOTAL = 0.07402  
FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+THICK	VAPOR
0.0500	0.5360	0.4140	0.1656	0.2484

HEXAGONAL CORNER CORRECTION FACTOR =1.2092

NUMBER OF HEAT PIPES = 84.9168

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 210

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 80.9  
AVG DELTA T ACROSS HEAT PIPE WALL = 13.1  
AVERAGE FUEL TEMPERATURE =1290.0  
MAXIMUM FUEL TEMPERATURE =1350.5

BURN FRACTION OF U235 =0.0648

FISSION DENSITY (FISSIONS/CM\*\*3) = 1.351E+21

FUEL SWELLING: VOLUME % = 5.29

" " dense fuel % = 6.22

REACTOR DIMENSIONS: METERS

	FUEL ELEMENT DIMENSIONS: MM
0.3210 CORE DIAMETER	21.04 WIDTH ACROSS HEX FLATS
0.3210 CORE HEIGHT	22.09 EQUIV. FUEL ELEMENT DIA
0.5510 REACTOR DIAMETER	21.54 EQUIV. FUEL REGION O.D.
0.5310 REACTOR HEIGHT	14.22 HEAT PIPE O.D.
0.1000 REFLECTOR THICKNESS	11.01 VAPOR DIAMETER
1.0000 PIPE LENGTH OUTSIDE REACTOR	95.24 VAPOR AREA: MM**2
1.4260 TOTAL HEAT PIPE LENGTH	
1.5310 OVERALL REACTOR+HEAT PIPE LENGTH	

REACTOR WEIGHTS: KILOGRAMS

152.4 FUEL: U235 MASS = 129.5  
251.5 REFLECTOR  
193.9 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 136.00  
33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)  
44.2 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
674.9 TOTAL REACTOR + HEAT PIPES

144.39 MW/M\*\*3: AVG POWR IN FUELSPACE 9.52 KW: POWER PER HEAT PIPE  
100.00 MW/M\*\*3: HTPIPE AXIAL HT FLUX 0.858 MW/M\*\*2: HTPIPE RAD HTFLX

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

TYPE NO OR STOP

80

\*\*\*\*\*  
PROP NO. 7 5-17-78 TYPE NEW INPUT: PAF=1. KHP=2 ...STOP  
PAF=1. STOP

4.000 (PR) REACTOR POWER:MW (KCORE) (1.2/UC+UC2) CORE =UC  
1250. (THP) HEAT PIPE TEMP:DEG K (KREF) (1.2/RE+RED) REFLECTOR =RED  
3650. (TIME) LIFETIME: DAYS (KHP) (1.2/NS+NO+W) HEAT PIPE =NO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR =NA  
10.0 (DAXL) AXIAL HT FLUX:MW/CM2 (IOPTN) (1.2) OPTION =2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING : DCORE(N) KREF(N) UNFT FEETA NPIPE ..STOP  
NPIPE=264 STOP

264 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG ZMIN DXMIN CORGAP ENDGAP  
0.150 0.005 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP

UC=0.004 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.585 0.943 \*

DCH = 0. 4.111 0.832 0.594 0.487 0.422 0.378 0.346 0.320 0.300

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1500 UNF =0.6570 VF =0.3430 DX =0.1000 DC =0.3954

REACTIVITY CHANGES: DELTA K

BURN = 0.05500 EXP = 0.01512 SAFE = 0.02000 TOTAL = 0.09012

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0.0500 0.4051 0.5449 0.2180 0.3269

HEXAGONAL CORNER CORRECTION FACTOR =1.3895

NUMBER OF HEAT PIPES = 101.8710

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 264

TEMPERATURE SUMMARY:DEGREE KELVIN

MAXIMUM FUEL DELTA T = 77.2

AVG DELTA T ACROSS HEAT PIPE WALL = 16.9

AVERAGE FUEL TEMPERATURE =1292.6

MAXIMUM FUEL TEMPERATURE =1352.5

BURN FRACTION OF U235 =0.0917

FISSION DENSITY (FISSIONS/CM\*\*3) = 1.910E+21

FUEL SWELLING:VOLUME % = 7.64

" " ,dense fuel % =8.99

REACTOR DIMENSIONS: METERS

FUEL ELEMENT DIMENSIONS: MM

0.3954	CORE DIAMETER	23.13	WIDTH ACROSS HEX FLATS
0.3954	CORE HEIGHT	24.29	EQUIV. FUEL ELEMENT DIA
0.6254	REACTOR DIAMETER	23.67	EQUIV. FUEL REGION O.D.
0.6054	REACTOR HEIGHT	17.93	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	13.89	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	151.46	VAPOR AREA: MM**2
1.5004	TOTAL HEAT PIPE LENGTH		
1.6054	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

215.5 FUEL: U235 MASS = 183.2

350.0 REFLECTOR

408.0 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 271.90

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

70.5 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
1076.9 TOTAL REACTOR + HEAT PIPES

204.17 MW/M\*\*3:AVG POWR IN FUELSPACE 15.15 KW:POWER PER HEAT PIPE

100.04 MW/M\*\*2:HTPIPE AXIAL HT FLUX 0.878 MW/M\*\*2:HTPIPE RAD HTFLX

\*\*\*\*\*  
TYPE GO OR STOP

GO

\*\*\*\*\*

PROR NO. 8 5-17-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
PR=0.2 THP=1425. STOP

0.200 (PR) REACTOR POWER: MW (KCORE) (1.2/UC+UC2) CORE =UC  
1425. (THP) HEAT PIPE TEMP: DEG K (KREF) (1.2/RE+RE2) REFLECTOR =RE2  
3650. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NE+NO+W) HEAT PIPE =NO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR =NA  
10.0 (DARL) AXIAL HT FLUX: KW/CM2 (IOPTN) (1.2) OPTION =2  
200. (DTENAY) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN: 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(M) XREF(M) UNFT FBETA NPIPE ...STOP  
NPIPE=84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA	VC	VCD	ALFA	PKAVG	BNIN	BYMIN	CORCAP	ENDCAP
0.150	0.004	0.050	0.600	1.500	0.050	0.080	0.015	0.005

TYPE: STOP: OR NEW CONSTANTS IE. VC=0, PKAVG=2, ETC ...STOP

VC=0.012 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.585 0.943 +.

DCH = 0. 0. 0.192 0.135 0.110 0.095 0.085 0.078 0.072 0.067

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1500 UNF =0.2743 VF =0.7257 DX =0.1000 DC =0.2237

REACTIVITY CHANGES: DELTA K

BUAN = 0.00718 EXP = 0.01764 SAFE = 0.02000 TOTAL = 0.04482

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+THICK	VAPOR
0.0500	0.8642	0.0858	0.0343	0.0515

HEXAGONAL CORNER CORRECTION FACTOR =1.0082

NUMBER OF HEAT PIPES = 35.3550

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 84.2

AVG DELTA T ACROSS HEAT PIPE WALL = 4.7

AVERAGE FUEL TEMPERATURE =1457.8

MAXIMUM FUEL TEMPERATURE =1516.2

BUAN FRACTION OF U235 =0.0120

FISSION DENSITY (FISSIONS/CM\*\*3) = 2.492E+20

FUEL SWELLING: VOLUME % = 3.27

" " , dense fuel % = 3.85

REACTOR DIMENSIONS: METERS

0.2237 CORE DIAMETER

0.2237 CORE HEIGHT

0.4537 REACTOR DIAMETER

0.4337 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR

1.3287 TOTAL HEAT PIPE LENGTH

1.4337 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

23.11 WIDTH ACROSS HEX PLATS

24.26 EQUIV. FUEL ELEMENT DIA

23.65 EQUIV. FUEL REGION O.D.

7.11 HEAT PIPE O.D.

5.51 VAPOR DIAMETER

23.80 VAPOR AREA: MM\*\*2

REACTOR WEIGHTS: KILOGRAMS

82.6 FUEL: U235 MASS = 70.2

149.7 REFLECTOR

18.1 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 13.60

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

19.8 SUPPORT STRUCTURE (7% OF REACTOR WT)

302.1 TOTAL REACTOR + HEAT PIPES

26.64 MW/CM\*\*3: AVG POWR IN FUELSPACE 2.38 KW: POWER PER HEAT PIPE

100.03 MW/CM\*\*2: HPIPE AXIAL HT FLUX 0.615 MW/CM\*\*2: HPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

GO

\*\*\*\*\*

PROP NO. 9 5-17-78 TYPE NEW INPUT: PR#1. KMR#2 ...STOP  
PR#0.4 STOP

0.400 (PR) REACTOR POWER: MW (KCORE) (1,2/UC:UO2) CORE #UC  
1425. (THR) HEAT PIPE TEMP: DEG K (KREF) (1,2/BE:BE0) REFLECTOR #BE0  
3650. (TIME) LIFETIME: DAYS (KMR) (1,2,3/NE:UO:U) HEAT PIPE #NO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI:NA) VAPOR #NA  
10.0 (GAXL) AXIAL HT FLUX: KW/CM2 (IORTH) (1,2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(N) KREF(N) UNIT FZETA NPIPE ..STOP  
NPIPE#84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG BMIN DMIN CORGAP ENDSAP  
0.150 0.012 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. VC=0. PKAVG=2. ETC ...STOP

STOP

SLD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.535 0.943 +.

DCH = 0. 0. 0.272 0.191 0.156 0.135 0.120 0.110 0.102 0.095

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO

BETA = 0.1500 VNF = 0.3295 VF = 0.6705 DX = 0.1000 DC = 0.2381

REACTIVITY CHANGES: DELTA K

BURN = 0.01288 EXP = 0.01764 SAFE = 0.02000 TOTAL = 0.05052

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+THICK VAPOR  
0.0500 0.7984 0.1516 0.0606 0.0910

HEXAGONAL CORNER CORRECTION FACTOR = 1.0258

NUMBER OF HEAT PIPES = 48.6978

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 115.9

AVG DELTA T ACROSS HEAT PIPE WALL = 8.8

AVERAGE FUEL TEMPERATURE = 1472.5

MAXIMUM FUEL TEMPERATURE = 1554.2

BURN FRACTION OF U235 = 0.0215

FISSION DENSITY (FISSIONS/CM\*\*3) = 4.474E+20

FUEL SWELLING: VOLUME % = 6.44

" " dense fuel % = 7.58

REACTOR DIMENSIONS: METERS

FUEL ELEMENT DIMENSIONS: MM

0.2381	CORE DIAMETER	24.59	WIDTH ACROSS HEX FLATS
0.2381	CORE HEIGHT	25.82	EQUIV. FUEL ELEMENT DIA
0.4681	REACTOR DIAMETER	25.17	EQUIV. FUEL REGION O.D.
0.4481	REACTOR HEIGHT	10.05	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	7.79	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	47.64	VAPOR AREA: MM**2
1.3431	TOTAL HEAT PIPE LENGTH		
1.4481	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

92.0 FUEL: U235 MASS = 78.2

162.1 REFLECTOR

36.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 27.21

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

22.7 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
346.3 TOTAL REACTOR + HEAT PIPES

47.83 MW/M\*\*3: AVG POWR IN FUELSpace 4.76 KW/POWER PER HEAT PIPE

99.96 MW/M\*\*2: HPIPE AXIAL HT FLUX 0.817 MW/M\*\*2: HPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP



\*\*\*\*\*  
 PAGE NO. 13 5-17-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
 STOP

1.000 (PR) REACTOR POWER: MW (CORE) (1.2/UC:UO2) CORE =UC  
 1425. (THP) HEAT PIPE TEMP: DEG K (KREF) (1.2/BE:ZEO) REFLECTOR =ZEO  
 3650. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NE:MO:W) HEAT PIPE =MO  
 1.00 (ELD) CORE L/D RATIO (KVAPOR) (1.2/LI:NA) VAPOR =NA  
 10.0 (DAXL) AXIAL HT FLUX: KW/CM2 (IOPTN) (1.2) OPTION =2  
 200. (DTMAX) MAX FUEL DELTA T: DEG K  
 1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE PT DESIGN; 2-SPECIFIED DESIGN  
 TYPE IN ANY OF FOLLOWING: DCORE(M) XREF(M) UNFT FBETA NPIPE ..STOP  
 NPIPE=162 STOP

162 (NPIPE) NO. OF HEAT PIPES  
 BETA VC VCD ALFA PHAVG BMIN DXMIN CORGAP ENDGAP  
 0.200 0.006 0.050 0.600 1.500 0.050 0.080 0.015 0.005  
 TYPE: STOP; OR NEW CONSTANTS IE. VC=0. PHAVG=2. ETC ...STOP

BETA=0.25 STOP

ELD INDEX = 3  
 VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000  
 DC = 0.196 0.203 0.230 0.260 0.299 0.352 0.437 0.585 0.943 +.  
 DCM = 0. 0. 1.391 0.383 0.276 0.227 0.197 0.177 0.162 0.150

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*  
 GO...ADJUST BETA FOR 10% SWELLING

BETA =0.2500 VNF =0.4804 VF =0.5196 DX =0.1000 DC =0.2904  
 REACTIVITY CHANGES: DELTA K  
 BURN = 0.02290 EXP = 0.01764 SAFE = 0.02000 TOTAL = 0.06054  
 FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.0500	0.6970	0.2530	0.1012	0.1513

HEXAGONAL CORNER CORRECTION FACTOR =1.0735

NUMBER OF HEAT PIPES = 70.8802

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 162

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.5

AVG DELTA T ACROSS HEAT PIPE WALL = 9.4

AVERAGE FUEL TEMPERATURE =1463.5

MAXIMUM FUEL TEMPERATURE =1526.6

BURN FRACTION OF U235 =0.0382

FISSION DENSITY (FISSIONS/CM\*\*3) = 7.017E+20

FUEL SWELLING: VOLUME % = 9.54

" " dense fuel % = 12.72

REACTOR DIMENSIONS: METERS

	FUEL ELEMENT DIMENSIONS: MM
0.2904 CORE DIAMETER	21.67 WIDTH ACROSS HEX FLATS
0.2904 CORE HEIGHT	22.75 EQUIV. FUEL ELEMENT DIA
0.5204 REACTOR DIAMETER	22.18 EQUIV. FUEL REGION D.D.
0.5004 REACTOR HEIGHT	11.44 HEAT PIPE O.D.
0.1000 REFLECTOR THICKNESS	8.86 VAPOR DIAMETER
1.0000 PIPE LENGTH OUTSIDE REACTOR	61.71 VAPOR AREA: MM**2
1.3954 TOTAL HEAT PIPE LENGTH	
1.5004 OVERALL REACTOR+HEAT PIPE LENGTH	

REACTOR WEIGHTS: KILOGRAMS

129.4 FUEL: U235 MASS = 110.0

216.1 REFLECTOR

94.9 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 67.98

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

33.1 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
 506.4 TOTAL REACTOR + HEAT PIPES

75.01 MW/M\*\*3: AVG POWR IN FUELSpace 6.17 KW/POWER PER HEAT PIPE  
 100.02 MW/M\*\*2: HTPIPE AXIAL HT FLUX 0.763 MW/M\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*  
 TYPE GO OR STOP

[illegible][illegible]

\*\*\*\*\*  
 PROB NO. 19 5-17-78 TYPE NEW INPUT: PPR1, KHP=2 ...STOP  
 STOP

4.000 (PR) REACTOR POWER:MW (KCORE) (1.2/UC+UB2) CORE =UC  
 1425. (TMR) HEAT PIPE TEMP:DEG K (KREF) (1.2/BE+BO) REFLECTOR =BO  
 3650. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NE+MO+M) HEAT PIPE =MO  
 1.00 (ELD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR =NA  
 10.0 (CAYL) AXIAL HT FLUX:MW/CM2 (IOPTN) (1.2) OPTION =2  
 200. (DTMAX) MAX FUEL DELTA T:DEG K  
 1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCORE(M) KREF(M) VNFT BETA NPIPE ..STOP  
 NPIPE=264 STOP

264 (NPIPE) NO. OF HEAT PIPES

BETA	VC	VCD	ALFA	PKAVG	BMIN	DXMIN	CORGAP	ENDGAP
0.600	0.004	0.050	0.600	1.500	0.050	0.080	0.015	0.005

TYPE: STOP: OR NEW CONSTANTS IE. VC=0. PKAVG=2. ETC ...STOP

BETA=0.56 STOP

ELD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.196 0.208 0.230 0.260 0.299 0.352 0.437 0.585 0.943 +.

DCH = 0. 0. 0. 0. 0. 1.512 0.567 0.416 0.344 0.300

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO...ADJUST BETA FOR 10% SWELLING

BETA =0.5600 VNF =0.7415 VF =0.2585 DX =0.1000 DC =0.4863

REACTIVITY CHANGES: DELTA K

BURN = 0.03924 EXP = 0.01764 SAFE = 0.02000 TOTAL = 0.07688

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.0500	0.5898	0.3602	0.1441	0.2161

HEXAGONAL CORNER CORRECTION FACTOR =1.1546

NUMBER OF HEAT PIPES = 127.4595

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 264

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 96.6

AVG DELTA T ACROSS HEAT PIPE WALL = 13.7

AVERAGE FUEL TEMPERATURE =1470.9

MAXIMUM FUEL TEMPERATURE =1542.2

BURN FRACTION OF U235 =0.0654

FISSION DENSITY (FISSIONS/CM\*\*3) = 7.054E+20

FUEL SWELLING: VOLUME % =10.05

" " dense fuel % = 22.84

REACTOR DIMENSIONS: METERS

0.4863 CORE DIAMETER

0.4863 CORE HEIGHT

0.7163 REACTOR DIAMETER

0.6963 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR 151.40

1.5913 TOTAL HEAT PIPE LENGTH

1.6963 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

28.45 WIDTH ACROSS HEX PLATS

29.87 EQUIV. FUEL ELEMENT DIA

29.11 EQUIV. FUEL REGION O.D.

17.92 HEAT PIPE O.D.

13.88 VAPOR DIAMETER

VAPOR AREA: MM\*\*2

REACTOR WEIGHTS: KILOGRAMS

302.0 FUEL: U235 MASS = 256.7

493.6 REFLECTOR

432.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 271.80

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG

98.3 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
 1349.4 TOTAL REACTOR + HEAT PIPES

75.40 MW/M\*\*3: AVG POWR IN FUELSPACE 15.15 MW: POWER PER HEAT PIPE

100.08 MW/M\*\*2: HPIPE AXIAL HT FLUX 0.714 MW/M\*\*2: HPIPE RAD HTFLX

\*\*\*\*\*



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\*\*\*\*\*  
 PROGRAM NO. 1 5-18-78 TYPE NEW INPUT: P=1, K=2 ... STOP  
 P=0.2 T=1425. TIME=3650. K=5 KREF=2 IOPTN=2 STOP

0.200 (PR) REACTOR POWER: MW (K=5) (1.2/UC:UC2) CORR =MO60UC2  
 1425. (T=) HEAT PIPE TEMP: DEG K (KREF) (1.2/SE:SE2) REFLECTOR =TEO  
 3650. (TIME) LIFETIME: DAYS (K=2) (1.2/NE:NO:W) HEAT PIPE =MO  
 1.00 (KLD) CORE L/D RATIO (K=2) (1.2/LI:LI) VAPOR =NA  
 10.0 (DAXL) AXIAL HT FLUX: KW/CM2 (IOPTN) (1.2) OPTION =2  
 200. (DTMAX) MAX FUEL DELTA T: DEG K  
 1.00 (WEL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: K=5 (M) KREF=2 (N) UNFT FSTA NPIPE ..STOP  
 NPIPE=84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAYS RMIN DXMIN CORGAP ENDSAP  
 0.100 0. 0.050 0.600 1.500 0.050 0.030 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0. PKAYS=2. ETC ...STOP

UC=0.012 VCD=0. STOP

FLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.207 0.142 0.115 0.099 0.088 0.081 0.074 0.070 0.066

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1000 UNF =0.1501 UF =0.8499 DX =0.1000 DC =0.3115

REACTIVITY CHANGES: DELTA K

BURN = 0.00464 EXP = 0.01274 SAFE = 0.02000 TOTAL = 0.03738

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.9558	0.0442	0.0177	0.0265

HEXAGONAL CORNER CORRECTION FACTOR =1.0020

NUMBER OF HEAT PIPES = 31.5836

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 75.2

AVG DELTA T ACROSS HEAT PIPE WALL = 3.4

AVERAGE FUEL TEMPERATURE =1453.4

MAXIMUM FUEL TEMPERATURE =1505.3

BURN FRACTION OF U235 =0.0077

FISSION DENSITY (FISSIONS/CM\*\*3) = 8.345E+19

FUEL SWELLING: VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3115 CORE DIAMETER

0.3115 CORE HEIGHT

0.5415 REACTOR DIAMETER

0.5215 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR

1.4165 TOTAL HEAT PIPE LENGTH

1.5215 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

32.18 WIDTH ACROSS HEX PLATS

33.79 EQUIV. FUEL ELEMENT DIA

33.79 EQUIV. FUEL REGION O.D.

7.11 HEAT PIPE O.D.

5.51 VAPOR DIAMETER

23.80 VAPOR AREA: MM\*\*2

REACTOR WEIGHTS: KILOGRAMS

214.3 FUEL: U235 MASS = 108.7

240.2 REFLECTOR

19.3 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 13.60

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

35.5 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
 542.2 TOTAL REACTOR + HEAT PIPES

8.92 MW/M\*\*3: AVG POWR IN FUELSPACE 2.38 MW: POWER PER HEAT PIPE

100.04 MW/M\*\*2: HTPIPE AXIAL HT FLUX 0.442 MW/M\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

PROG NO. 2 5-18-78 TYPE NEW INPUT: PR#1, KHP#2 ...STOP  
PR#0.4 STOP

0.400 (PR) REACTOR POWER:MW (KCORE) (1.2/UC:UO2) CORE #NO60UO2  
1425. (THP) HEAT PIPE TEMP:DEG K (KREF) (1.2/BE:PEO) REFLECTOR #PEO  
3650. (TIME) LIFETIME:DAY (KHP) (1.2/3/NB:HO:W) HEAT PIPE #MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI:NA) VAPOR #NA  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (NPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE 1 1-CODE PT DESIGN 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING 1 DCORE(M) XREF(M) UNFT FEET NPIPE ..STOP  
NPIPE#84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC UCD ALFA PKAVG BMIN DXMIN CORCAP ENDCAP  
0.100 0.012 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP

STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.

DCM = 0. 0.293 0.201 0.163 0.140 0.125 0.114 0.105 0.098 0.093

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO

BETA =0.1000 UNF =0.1848 UF =0.8152 DX =0.1000 DC =0.3214

REACTIVITY CHANGES: DELTA K

BURN = 0.00281 EXP = 0.01274 SAFE = 0.02000 TOTAL = 0.04155

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.9167 0.0833 0.0333 0.0500

HEXAGONAL CORNER CORRECTION FACTOR =1.0070

NUMBER OF HEAT PIPES = 46.5570

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY:DEGREE KELVIN

MAXIMUM FUEL DELTA T = 110.8

AVG DELTA T ACROSS HEAT PIPE WALL = 6.5

AVERAGE FUEL TEMPERATURE =1468.5

MAXIMUM FUEL TEMPERATURE =1545.7

BURN FRACTION OF U235 =0.0147

FISSION DENSITY (FISSIONS/CM\*\*3) = 1.585E+20

FUEL SWELLING:VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3214	CORE DIAMETER	33.19	WIDTH ACROSS HEX PLATE
0.3214	CORE HEIGHT	34.85	EQUIV. FUEL ELEMENT DIA
0.5514	REACTOR DIAMETER	34.85	EQUIV. FUEL REGION O.D.
0.5314	REACTOR HEIGHT	10.06	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	7.79	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	47.66	VAPOR AREA: MM**2
1.4264	TOTAL HEAT PIPE LENGTH		
1.5314	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

225.6 FUEL: U235 MASS = 114.4  
251.9 REFLECTOR  
38.8 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 27.22  
33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)  
38.5 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
587.9 TOTAL REACTOR + HEAT PIPES

16.94 MW/M\*\*3:AVG POWR IN FUELSPACE 4.76 KW/POWER PER HEAT PIPE  
99.92 MW/M\*\*2:HTPIPE AXIAL HT FLUX 0.605 MW/M\*\*2:HTPIPE RAD HTFLX

\*\*\*\*\*  
TYPE GO OR STOP

80

PROG NO. 3 5-18-78 TYPE NEW INPUT: PR=1. KMP=2 ...STOP  
PR=0.7 STOP

0.700 (PR) REACTOR POWER:MW (KCORE) (1.2/UC+UC2) CORE #MO50UC2  
1425. (TMP) HEAT PIPE TEMP:DEG K (KREF) (1.2/SE+SE2) REFLECTOR #SE2  
3650. (TIME) LIFETIME: DAYS (KMP) (1.2/3/NS+MO+M) HEAT PIPE #MO  
1.00 (FLD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR #NA  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE 1 1-CODE AT DESIGN: 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING: DCOE(M) XREF(M) UNFT FBETA NPIPE ...STOP  
NPIPE=120 STOP

120 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAYS BMIN DXMIN CORCOR ENDCOR  
0.100 0.012 0. 0.600 1.500 0.050 0.030 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAYS=2. ETC ...STOP  
UC=0.003 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCM = 0. 0.380 0.263 0.214 0.184 0.165 0.150 0.139 0.130 0.122

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦  
80

BETA = 0.1000 UNF = 0.2264 UF = 0.7736 DX = 0.1000 DC = 0.3348

REACTIVITY CHANGES: DELTA K

BURN = 0.01435 EXP = 0.01274 SAFE = 0.02000 TOTAL = 0.04709

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.8665 0.1335 0.0534 0.0801

HEXAGONAL CORNER CORRECTION FACTOR = 1.0180

NUMBER OF HEAT PIPES = 61.1476

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 120

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 101.9

Avg DELTA T ACROSS HEAT PIPE WALL = 7.7

AVERAGE FUEL TEMPERATURE = 1466.7

MAXIMUM FUEL TEMPERATURE = 1538.4

BURN FRACTION OF U235 = 0.0239

FISSION DENSITY (FISSIONS/CM\*\*3) = 2.584E+20

FUEL ENRICHING VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3348 CORE DIAMETER

0.3348 CORE HEIGHT

0.5648 REACTOR DIAMETER

0.5448 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR

1.4398 TOTAL HEAT PIPE LENGTH

1.5448 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

29.00 WIDTH ACROSS HEX PLATE

30.44 EQUIV. FUEL ELEMENT DIA

30.44 EQUIV. FUEL REGION O.D.

11.12 HEAT PIPE O.D.

8.62 VAPOR DIAMETER

58.31 VAPOR AREA: MM\*\*2

REACTOR WEIGHTS: KILOGRAMS

242.3 FUEL: U235 MASS = 122.8

268.5 REFLECTOR

68.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 47.58

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

42.9 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
655.2 TOTAL REACTOR + HEAT PIPES

27.62 MW/M\*\*3: AVG POWR IN FUELSPACE 5.83 MW: POWER PER HEAT PIPE

100.05 MW/M\*\*3: HTPIPE AXIAL HT FLUX 0.644 MW/M\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*  
TYPE GO OR STOP

GO

\*\*\*\*\*

PROB NO. 4 5-12-78 TYPE NEW INPUT: PR=1. KHP=2 ...STOP  
PR=1. STOP

1.000 (PR) REACTOR POWER:MW (KCORE) (1.2/UC:UO2) CORE =MO60UO2  
1425. (THP) HEAT PIPE TEMP:DEG K (KREF) (1.2/BE:BEQ) REFLECTOR =BEO  
3650. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NB:MO:W) HEAT PIPE =MO  
1.00 (ELD) CORE L/D RATIO (KVAPOR) (1.2/LI:NA) VAPOR =NA  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1.2) OPTION =2  
200. (DTFMK) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(M) XREF(M) UNFT FBETA NPIPE ..STOP  
NPIPE=162 STOP

162 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAYS BMIN DXMIN CORRAF ENDSAP  
0.100 0.008 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAYS=2. ETC ...STOP

UC=0.006 STOP

ELD INDEX = 3

UNE = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.

DCH = 0. 0.449 0.313 0.255 0.220 0.197 0.179 0.166 0.155 0.146

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1000 UNE =0.2637 UF =0.7370 DX =0.1000 DC =0.3482

REACTIVITY CHANGES: DELTA K

BURN = 0.01915 EXP = 0.01274 SAFE = 0.02000 TOTAL = 0.05189

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.8238 0.1762 0.0705 0.1057

HEXAGONAL CORNER CORRECTION FACTOR =1.0315

NUMBER OF HEAT PIPES = 71.2219

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 162

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.9

AVG DELTA T ACROSS HEAT PIPE WALL = 7.8

AVERAGE FUEL TEMPERATURE =1462.1

MAXIMUM FUEL TEMPERATURE =1524.7

BURN FRACTION OF U235 =0.0319

FISSION DENSITY (FISSIONS/CM\*\*3) = 3.447E+20

FUEL SWELLING: VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3482	CORE DIAMETER	25.97	WIDTH ACROSS HEX FLATS
0.3482	CORE HEIGHT	27.27	EQUIV. FUEL ELEMENT DIA
0.5782	REACTOR DIAMETER	27.27	EQUIV. FUEL REGION O.D.
0.5582	REACTOR HEIGHT	11.45	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	8.87	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	61.74	VAPOR AREA: MM**2
1.4532	TOTAL HEAT PIPE LENGTH		
1.5582	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

259.4 FUEL: U235 MASS = 131.5  
285.5 REFLECTOR  
98.8 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 68.02  
33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)  
47.4 SUPPORT STRUCTURE (7% OF REACTOR WT)

724.1 TOTAL REACTOR + HEAT PIPES

36.84 MW/M\*\*3: AVG POWR IN FUELSPACE 6.17 KW: POWER PER HEAT PIPE  
99.98 MW/M\*\*2: HTRPIPE AXIAL HT FLUX 0.637 MW/M\*\*2: HTRPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

30

PROB NO. 5 5-18-78 TYPE NEW INPUT: PR#1. KWP#2 ...STOP  
PR#2. STOP

2.000 (PR) REACTOR POWER: MW (KCORE) (1.2/UC:UD2) CORE #NO60UD2  
1425. (THR) HEAT PIPE TEMP: DEG K (KREF) (1.2/RE:DEO) REFLECTOR #2EO  
3650. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NB:MO:W) HEAT PIPE #MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI:NA) VAPOR #NA  
10.0 (DAXL) AXIAL HT FLUX: KW/CM2 (IOPTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN: 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCORE(M) XREF(M) UNFT FBETA NPIPE ..STOP  
NPIPE#210 STOP

210 (NPIPE) NO. OF HEAT PIPES

BETA	UC	UCD	ALFA	PKAVG	BMIN	DXMIN	CORCAP	ENDCAP
0.100	0.005	0.	0.600	1.500	0.050	0.030	0.015	0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAVG#2. ETC ...STOP

UC=0.005 STOP

SLD INDEX = 3

UNA = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.

DCH = 0. 0.632 0.442 0.360 0.311 0.278 0.253 0.234 0.219 0.207

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

30

BETA = 0.1000 UNA = 0.3566 UN = 0.6434 DX = 0.1000 DC = 0.3893

REACTIVITY CHANGES: DELTA K

BURN = 0.03138 EXP = 0.01274 SAFE = 0.02000 TOTAL = 0.06412

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.7185	0.2815	0.1126	0.1689

HEXAGONAL CORNER CORRECTION FACTOR = 1.0325

NUMBER OF HEAT PIPES = 92.2398

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 210

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.8

AVG DELTA T ACROSS HEAT PIPE WALL = 10.8

AVERAGE FUEL TEMPERATURE = 1465.1

MAXIMUM FUEL TEMPERATURE = 1529.0

BURN FRACTION OF U235 = 0.0523

FISSION DENSITY (FISSIONS/CM\*\*3) = 5.648E+20

FUEL SWELLING: VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3893 CORE DIAMETER

0.3893 CORE HEIGHT

0.6193 REACTOR DIAMETER

0.5993 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR

1.4943 TOTAL HEAT PIPE LENGTH

1.5993 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

25.52 WIDTH ACROSS HEX FLATS

26.80 EQUIV. FUEL ELEMENT DIA

26.80 EQUIV. FUEL REGION O.D.

14.32 HEAT PIPE O.D.

11.01 VAPOR DIAMETER

95.26 VAPOR AREA: MM\*\*2

REACTOR WEIGHTS: KILOGRAMS

316.6 FUEL: U235 MASS = 160.5

341.3 REFLECTOR

203.3 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 136.03

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

62.6 SUPPORT STRUCTURE (7% OF REACTOR WT)

956.7 TOTAL REACTOR + HEAT PIPES

40.37 MW/M\*\*3: AVG POWR IN FUELSpace 9.52 KW: POWER PER HEAT PIPE

99.93 MW/M\*\*2: HTRPIPE AXIAL HT FLUX 0.707 MW/M\*\*2: HTRPIPE RAD HTFLX

TYPE GO OR STOP

30

\*\*\*\*\*

PROP NO. 5 5-18-78 TYPE NEW INPUT: PR#1, KH#2 ...STOP  
PR#4, STOP

4.000 (PR) REACTOR POWER:MW (KCORE) (1.2/UC+UC2) CORE #N050002  
1425. (THR) HEAT PIPE TEMP:DEG K (KREF) (1.2/8+800) REFLECTOR #800  
3450. (TIME) LIFETIME: DAYS (KHE) (1.2/3+NA+NH) HEAT PIPE #N0  
1.00 (ELD) CORE L/D RATIO (KVAPOR) (1.2/LI+LA) VAPOR #NA  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IDPTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE : 1-CODE AT DESIGN: 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING : DCORE(M) KREF(M) UNFT FBETA NPIPE ..STOP  
NPIPE#266 STOP

266 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKANG EMIN DXMIN CORGAP ENDGAP  
0.100 0.004 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0, PKANG=2, ETC ...STOP

STOP

ELD INDEX = 3

UNE = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.890 0.624 0.508 0.439 0.392 0.358 0.331 0.310 0.292

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*  
GO

BETA = 0.1000 UNE = 0.4720 UE = 0.5280 DX = 0.1000 DC = 0.4554

REACTIVITY CHANGES: DELTA K

BURN = 0.04777 EXP = 0.01274 SAFE = 0.02000 TOTAL = 0.08051

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.5890 0.4110 0.1644 0.2466

HEXAGONAL CORNER CORRECTION FACTOR = 1.1840

NUMBER OF HEAT PIPES = 115.0234

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 266

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 86.5

AVG DELTA T ACROSS HEAT PIPE WALL = 14.6

AVERAGE FUEL TEMPERATURE = 1468.4

MAXIMUM FUEL TEMPERATURE = 1533.3

BURN FRACTION OF U235 = 0.0796

FISSION DENSITY (FISSIONS/CM\*\*3) = 8.598E+20

FUEL SWELLING: VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.4554 CORE DIAMETER

0.4554 CORE HEIGHT

0.6854 REACTOR DIAMETER

0.6654 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR 150.41

1.5604 TOTAL HEAT PIPE LENGTH

1.6654 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

26.54 WIDTH ACROSS HEX FLATS

27.87 EQUIV. FUEL ELEMENT DIA

27.87 EQUIV. FUEL REGION O.D.

17.87 HEAT PIPE O.D.

13.84 VAPOR DIAMETER

VAPOR AREA: MM\*\*2

REACTOR WEIGHTS: KILOGRAMS

416.0 FUEL: U235 MASS = 210.9

442.0 REFLECTOR

424.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 272.05

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

92.1 SUPPORT STRUCTURE (7% OF REACTOR WT)

1407.5 TOTAL REACTOR + HEAT PIPES

91.91 MW/M\*\*3: AVG POWR IN FUELSPACE 15.04 KW: POWER PER HEAT PIPE

99.99 MW/M\*\*2: HPIPE AXIAL HT FLUX 0.760 MW/M\*\*2: HPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

[illegible]

TYPE GO OR STOP

\*\*\*\*\*  
PAGE NO. 10 5-18-78 TYPE NEW INPUT: PR=1. KHP=2 ...STOP  
PR=0.4 STOP

0.400 (PR) REACTOR POWER, MW (KCORE) (1.2/UC:UC2) CORE =NO60002  
1600. (THR) HEAT PIPE TEMP, DEG K (KREF) (1.2/EE:EE2) REFLECTOR =REF  
3650. (TIME) LIFETIME, DAYS (KHP) (1.2/3/NE:NO:W) HEAT PIPE =NO  
1.00 (FLD) CORE L/D RATIO (KVAPOR) (1.2/LI:NA) VAPOR =LI  
10.0 (DAXL) AXIAL HT FLUX, KW/CM2 (IOFTN) (1.2) OPTION =2  
200. (DTFMAX) MAX FUEL DELTA T, DEG K  
1.00 (NPL1) PIPE EXTENSION, M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN: 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING : DCORE(M) XREF(M) UNFT FBETA NPIPE ...STOP  
NPIPE=84 STOP

84 (NPIPE) NO. OF HEAT PIPES  
BETA UC VCD ALFA PKAWG BMIN DXMIN CORGAP ENRGAP  
0.100 0.012 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAWG=2. ETC ...STOP  
STOP

ELD INDEX = 3  
UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000  
DC = 0.239 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.  
DCH = 0. 0.293 0.201 0.163 0.140 0.125 0.114 0.105 0.098 0.093  
◆ ◆ ◆ ◆ ◆ ◆ ◆ ◆ ◆ ◆ TYPE GO OR START OVER ◆ ◆ ◆ ◆ ◆ ◆ ◆ ◆ ◆ ◆

GO  
BETA =0.1000 UNF =0.1843 UF =0.8152 DX =0.1000 DC =0.3214

REACTIVITY CHANGES: DELTA K  
BURN = 0.00881 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.04337  
FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.9167	0.0833	0.0333	0.0500

HEXAGONAL CORNER CORRECTION FACTOR =1.0070

NUMBER OF HEAT PIPES = 46.5570

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 110.8

AVG DELTA T ACROSS HEAT PIPE WALL = 6.5

AVERAGE FUEL TEMPERATURE =1643.5

MAXIMUM FUEL TEMPERATURE =1720.7

BURN FRACTION OF U235 =0.0147

FISSION DENSITY (FISSIONS/CM\*\*3) = 1.58E+20

FUEL SWELLING VOLUME % = 0.

REACTOR DIMENSIONS: METERS

REACTOR DIMENSIONS: METERS		FUEL ELEMENT DIMENSIONS: MM	
0.3214	CORE DIAMETER	33.19	WIDTH ACROSS HEX FLATS
0.3214	CORE HEIGHT	34.85	EQUIV. FUEL ELEMENT DIA
0.5514	REACTOR DIAMETER	34.85	EQUIV. FUEL REGION O.D.
0.5314	REACTOR HEIGHT	10.06	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	7.79	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	47.66	VAPOR AREA: MM**2
1.4264	TOTAL HEAT PIPE LENGTH		
1.5314	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

225.6	FUEL: U235 MASS = 114.4
251.9	REFLECTOR
38.6	HEAT PIPES: WT/UNIT LENGTH (KG/M) = 27.22
33.0	CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)
38.5	SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
587.9 TOTAL REACTOR + HEAT PIPES

16.94 MW/M\*\*3: AVG POWR IN FUELSpace 4.76 KW/POWER PER HEAT PIPE  
99.92 MW/M\*\*2: HTPIPE AXIAL HT FLUX 0.605 MW/M\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP



GO

\*\*\*\*\*  
 PROB NO. 11 5-18-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
 PR=0.7 STOP

0.700 (PR) REACTOR POWER:MW (KCORE) (1,2/UC:UD2) CORE #MO60UD2  
 1600. (THP) HEAT PIPE TEMP:DEG K (KREF) (1,2/BE:REO) REFLECTOR #REO  
 3650. (TIME) LIFETIME:DAYS (KHP) (1,2,3/NE:MO:W) HEAT PIPE #MO  
 1.00 (ELD) CORE L/D RATIO (KVAPOR) (1,2/LI:NA) VAPOR #LI  
 10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1,2) OPTION #2  
 200. (DTMAX) MAX FUEL DELTA T:DEG K  
 1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(M) XREF(M) UNFT FBETA NPIPE ...STOP  
 NPIPE=120 STOP

120 (NPIPE) NO. OF HEAT PIPES

BETA	UC	UCD	ALFA	PKAVG	ZMIN	DXMIN	CORRSP	ENDGAP
0.100	0.012	0.	0.600	1.500	0.050	0.080	0.015	0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0, PKAVG=2, ETC ...STOP

UC=0.008 STOP

ELD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.

DCH = 0. 0.380 0.263 0.214 0.184 0.165 0.150 0.139 0.130 0.122

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO

BETA =0.1000 UNF =0.2264 UF =0.7736 DX =0.1000 DC =0.3348

REACTIVITY CHANGES: DELTA K

BURN = 0.01435 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.04891

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.8665	0.1335	0.0534	0.0801

HEXAGONAL CORNER CORRECTION FACTOR =1.0180

NUMBER OF HEAT PIPES = 61.1476

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 120

TEMPERATURE SUMMARY:DEGREE KELVIN

MAXIMUM FUEL DELTA T = 101.9

AVG DELTA T ACROSS HEAT PIPE WALL = 7.7

AVERAGE FUEL TEMPERATURE =1641.7

MAXIMUM FUEL TEMPERATURE =1713.4

BURN FRACTION OF U235 =0.0239

FISSION DENSITY (FISSIONS/CM\*\*3) = 2.584E+20

FUEL SWELLING:VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3348	CORE DIAMETER
0.3348	CORE HEIGHT
0.5648	REACTOR DIAMETER
0.5448	REACTOR HEIGHT
0.1000	REFLECTOR THICKNESS
1.0000	PIPE LENGTH OUTSIDE REACTOR
1.4398	TOTAL HEAT PIPE LENGTH
1.5448	OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

29.00	WIDTH ACROSS HEX FLATS
30.44	EQUIV. FUEL ELEMENT DIA
30.44	EQUIV. FUEL REGION O.D.
11.12	HEAT PIPE O.D.
8.62	VAPOR DIAMETER
58.31	VAPOR AREA: MM**2

REACTOR WEIGHTS: KILOGRAMS

242.3	FUEL: U235 MASS = 122.8
268.5	REFLECTOR
68.5	HEAT PIPES: WT/UNIT LENGTH (KG/M) = 47.58
33.0	CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)
42.9	SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
 655.2 TOTAL REACTOR + HEAT PIPES

27.62	MW/M**3:AVG POWR IN FUELSpace	5.83	KW:POWER PER HEAT PIPE
100.05	MW/M**2:HTPIPE AXIAL HT FLUX	0.644	MW/M**2:HTPIPE RAD HTFLX

\*\*\*\*\*

PROB NO. 12 5-18-78 TYPE NEW INPUT: PR=1, KHP=2 ...ETOP  
PR=1. STOP

1.000 (PR) REACTOR POWER:MW (KCORE) (1.2/UC+UC2) CORE =MO60UC2  
1600. (THP) HEAT PIPE TEMP:DEG K (KREF) (1.2/RE+REO) REFLECTOR =REO  
3650. (TIME) LIFETIME:DAY (KHP) (1.2/3/NE+NO+W) HEAT PIPE =MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1.2/LI+NA) VAPOR =LI  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1.2) OPTION =2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HALI) PIPE EXTENSION:M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN: 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING : DCORE(M) XREF(M) UNFT FBETA NPIPE ..STOP  
NPIPE=162 STOP

162 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG ZMIN DXMIN CORGAP ENDGAP  
0.100 0.008 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0, PKAVG=2, ETC ...STOP

UC=0.006 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.449 0.313 0.255 0.220 0.197 0.179 0.166 0.155 0.146

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1000 UNF =0.2630 UF =0.7370 DX =0.1000 DC =0.3482

REACTIVITY CHANGES: DELTA K

BURN = 0.01915 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.05371

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.8238 0.1762 0.0705 0.1057

HEXAGONAL CORNER CORRECTION FACTOR =1.0315

NUMBER OF HEAT PIPES = 71.2219

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 162

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.9

AVG DELTA T ACROSS HEAT PIPE WALL = 7.8

AVERAGE FUEL TEMPERATURE =1637.1

MAXIMUM FUEL TEMPERATURE =1699.7

BURN FRACTION OF U235 =0.0319

FISSION DENSITY (FISSION/CM\*\*3) = 3.447E+20

FUEL SWELLING: VOLUME % = 0.

REACTOR DIMENSIONS: METERS

0.3482	CORE DIAMETER	25.97	WIDTH ACROSS HEX FLATS
0.3482	CORE HEIGHT	27.27	EQUIV. FUEL ELEMENT DIA
0.5782	REACTOR DIAMETER	27.27	EQUIV. FUEL REGION O.D.
0.5582	REACTOR HEIGHT	11.45	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	8.87	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	61.74	VAPOR AREA: MM**2
1.4532	TOTAL HEAT PIPE LENGTH		
1.5582	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

259.4	FUEL: U235 MASS = 131.5
285.5	REFLECTOR
98.8	HEAT PIPES: WT/UNIT LENGTH (KG/M) = 68.02
33.0	CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)
47.4	SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
724.1 TOTAL REACTOR + HEAT PIPES

36.84 MW/M\*\*3:AVG POWR IN FUELESPACE 6.17 KW:POWER PER HEAT PIPE  
99.98 MW/M\*\*2:HTPIPE AXIAL HT FLUX 0.637 MW/M\*\*2:HTPIPE RAD HTFLX

\*\*\*\*\*  
TYPE GO OR STOP

PROB NO. 2 8-2-78  
STOP

TYPE NEW INPUT: 78=1. KMP? ...STOP

ORIGINAL PAGE IS  
OF POOR QUALITY

1.650 (PR) REACTOR POWER,MW (KCORE) (1,2/UC,UC2) CORE =MO60UC2  
1600. (THP) HEAT PIPE TEMP,DEG K (KREF) (1,2/BE,BED) REFLECTOR =BED  
3650. (TIME) LIFETIME,DAYS (KMP) (1,2/3/NB,MO,W) HEAT PIPE =MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI,NA) VAPOR =LI  
10.0 (DAXL) AXIAL HT FLUX,KW/CM2 (IOPTN) (1,2) OPTION =2  
200. (DTFMAX) MAX FUEL DELTA T,DEG K  
1.00 (HPL1) PIPE EXTENSION,M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCOPE(M) XREF(M) VNPT FBETA NPIPE ..STOP  
NPIPE=162 STOP

162 (NPIPE) NO. OF HEAT PIPES

BETA	VC	VCD	ALFA	PKAVG	BMIN	DXMIN	CORCAP	ENDCAP
0.100	0.	0.050	0.600	1.500	0.050	0.080	0.015	0.005

TYPE: STOP; OR NEW CONSTANTS IE. VC=0. PKAVG=2. ETC ...STOP

BETA=0.1 VC=0.006 VCD=0. STOP

SLD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.

DCH = 0. 0.577 0.402 0.327 0.283 0.252 0.230 0.213 0.199 0.188

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR RESTART ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO...PT DESIGN FOR PRES LAYTON AND HARRER

BETA =0.1000 VNF =0.3286 VF =0.6714 DX =0.1000 DC =0.3759

REACTIVITY CHANGES: DELTA K

BURN = 0.02756 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.06212

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.7505	0.2495	0.0999	0.1497

HEXAGONAL CORNER CORRECTION FACTOR =1.0642

NUMBER OF HEAT PIPES = 86.1780

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 162

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 106.4

AVG DELTA T ACROSS HEAT PIPE WALL = 12.0

AVERAGE FUEL TEMPERATURE =1647.4

MAXIMUM FUEL TEMPERATURE =1724.3

BURN FRACTION OF U235 =0.0459

FISSION DENSITY (FISSIONS/CM\*\*3) = 4.960E+20

FUEL SWELLING,VOLUME % = 0.43\*3 = 1.29

REACTOR DIMENSIONS: METERS

FUEL ELEMENT DIMENSIONS: MM

0.3759	CORE DIAMETER	28.04	WIDTH ACROSS HEX FLATS
0.3759	CORE HEIGHT	29.44	EQUIV. FUEL ELEMENT DIA
0.6059	REACTOR DIAMETER	29.44	EQUIV. FUEL REGION O.D.
0.5859	REACTOR HEIGHT	14.71	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	11.39	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	101.90	VAPOR AREA: MM**2
1.4809	TOTAL HEAT PIPE LENGTH		
1.5859	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

297.3 FUEL, U235 MASS = 150.8

322.4 REFLECTOR

166.2 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 112.25

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

57.3 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
876.3 TOTAL REACTOR + HEAT PIPES

53.04 MW/M\*\*3,AVG POWR IN FUELSPACE 10.19 KW/POWER PER HEAT PIPE

99.96 MW/M\*\*2,HTPIPE AXIAL HT FLUX 0.757 MW/M\*\*2,HTPIPE RAD HTFLX

\*\*\*\*\*

PROB NO. 3 8-2-78 TYPE NEW INPUT: PR=1. KHP=2 ...STOP  
STOP

1.650 (PR) REACTOR POWER: MW (KCORE) (1,2/UC;UD2) CORE =MO60UD2  
1600. (THP) HEAT PIPE TEMP: DEG K (KREF) (1,2/BE;BED) REFLECTOR =BED  
3650. (TIME) LIFETIME: DAYS (KHP) (1,2,3/NE;MO;W) HEAT PIPE =MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI;NA) VAPOR =LI  
10.0 (MAXL) AXIAL HT FLUX: KW/CM2 (LOPTN) (1,2) OPTION =2  
200. (DTFMAX) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCORE(M) XREF(M) VNFT FBETA NPIPE ..STOP  
NPIPE=210.\\. STOP

210 (NPIPE) NO. OF HEAT PIPES

BETA	UC	VCD	ALFA	PKAVG	BMIN	DXMIN	CORCAP	ENDCAP
0.100	0.006	0.	0.600	1.500	0.050	0.080	0.015	0.005

TYPE: STOP; OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP

UC=0.005 STOP

SLD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.574 0.401 0.327 0.282 0.252 0.230 0.213 0.199 0.188

\*\*\*\*\* TYPE GO OR RESTART \*\*\*\*\*  
GO...PT DESIGN FOR PRES LAYTON AND HARPER; NPIPE=210

BETA =0.1000 VNF =0.3280 VF =0.6720 DX =0.1000 DC =0.3756

REACTIVITY CHANGES: DELTA K

BURN = 0.02760 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.06216

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.7504	0.2496	0.0998	0.1497

HEXAGONAL CORNER CORRECTION FACTOR =1.0643

NUMBER OF HEAT PIPES = 86.2171

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 210

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 82.1

AVG DELTA T ACROSS HEAT PIPE WALL = 9.2

AVERAGE FUEL TEMPERATURE =1636.6

MAXIMUM FUEL TEMPERATURE =1696.0

BURN FRACTION OF U235 =0.0460

FISSION DENSITY (FISSIONS/CM\*\*3) = 4.968E+20

FUEL SWELLING: VOLUME % = 0.37 x3 = 1.1

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REACTOR DIMENSIONS: METERS

REACTOR DIMENSIONS: METERS		FUEL ELEMENT DIMENSIONS: MM	
0.3756	CORE DIAMETER	24.62	WIDTH ACROSS HEX FLATS
0.3756	CORE HEIGHT	25.85	EQIV. FUEL ELEMENT DIA
0.6056	REACTOR DIAMETER	25.85	EQIV. FUEL REGION O.D.
0.5856	REACTOR HEIGHT	12.92	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	10.00	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	78.60	VAPOR AREA: MM**2
1.4806	TOTAL HEAT PIPE LENGTH		
1.5856	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

297.0 FUEL; U235 MASS = 150.6

322.1 REFLECTOR

166.2 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 112.24

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

57.3 SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
875.5 TOTAL REACTOR + HEAT PIPES

53.11 MW/M\*\*3; AVG POWR IN FUELSPACE 7.86 KW; POWER PER HEAT PIPE

99.96 MW/M\*\*2; HPIPE AXIAL HT FLUX 0.666 MW/M\*\*2; HPIPE RAD HTFLX

\*\*\*\*\*

PROB NO. 13 5-18-78 TYPE NEW INPUT: PR=1, KMR=2 ...STOP  
PR=2. STOP

2.000 (PR) REACTOR POWER: MW (KCORE) (1.2/UC+UC2) CORE #M060U02  
1600. (THP) HEAT PIPE TEMP: DEG K (KREF) (1.2/BE+BE2) REFLECTOR #B02  
3650. (TIME) LIFETIME: DAYS (KMR) (1.2/3/NS+NO+U) HEAT PIPE #M0  
1.00 (ELD) CORE LTD RATIO (KVAPOR) (1.2/LI+NA) VAPOR #LI  
10.0 (CARL) AXIAL HT FLUX: KW/CM<sup>2</sup> (IORTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T: DEG K  
1.00 (HPL1) PIPE EXTENSION: M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN; 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING: DCORE(N) XREF(N) UNIT FBETA NPIPE ...STOP  
NPIPE=210 STOP

210 (NPIPE) NO. OF HEAT PIPES  
BETA UC UCD ALFA PKANG BMIN DYMIN CORGAP ENDGAP  
0.100 0.006 0. 0.600 1.500 0.050 0.080 0.015 0.005  
TYPE: STOP; OR NEW CONSTANTS IE. UC=0. PKANG=2. ETC ...STOP  
UC=0.005 STOP

SLD INDEX = 3  
MNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000  
DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.  
DCH = 0. 0.632 0.442 0.360 0.311 0.278 0.253 0.234 0.219 0.207

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GO

BETA = 0.1000 MNF = 0.3566 MF = 0.6434 DX = 0.1000 DC = 0.3893  
REACTIVITY CHANGES: DELTA K

BURN = 0.03138 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.06594

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.7185	0.2815	0.1126	0.1689

HEXAGONAL CORNER CORRECTION FACTOR = 1.0825

NUMBER OF HEAT PIPES = 92.2398

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 210

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.8

AVG DELTA T ACROSS HEAT PIPE WALL = 10.8

AVERAGE FUEL TEMPERATURE = 1640.1

MAXIMUM FUEL TEMPERATURE = 1704.0

BURN FRACTION OF U235 = 0.0523

FISSION DENSITY (FISSIONS/CM<sup>3</sup>) = 5.648E+20

FUEL SWELLING: VOLUME % = 0.

REACTOR DIMENSIONS: METERS

REACTOR DIMENSIONS: METERS		FUEL ELEMENT DIMENSIONS: MM	
0.3893	CORE DIAMETER	25.52	WIDTH ACROSS HEX PLATE
0.3893	CORE HEIGHT	26.80	EQUIV. FUEL ELEMENT DIA
0.6193	REACTOR DIAMETER	26.80	EQUIV. FUEL REGION O.D.
0.5993	REACTOR HEIGHT	14.22	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	11.01	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	95.26	VAPOR AREA: MM <sup>2</sup>
1.4943	TOTAL HEAT PIPE LENGTH		
1.5993	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

316.6	FUEL: U235 MASS = 160.5
341.3	REFLECTOR
203.3	HEAT PIPES: WT/UNIT LENGTH (KG/M) = 136.03
33.0	CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)
62.6	SUPPORT STRUCTURE (7% OF REACTOR WT)

-----  
956.7 TOTAL REACTOR + HEAT PIPES

60.37 MW/M<sup>3</sup> + 3. AVG FOUR IN FUEL SPACE 9.52 MW: POWER PER HEAT PIPE  
99.99 MW/M<sup>2</sup> + 2. HTPIPE AXIAL HT FLUX 0.707 MW/M<sup>2</sup> + 2. HTPIPE RAD HTFLX

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TYPE GO OR STOP

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NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCOFF(M) XREF(M) UNFT FBETA NPIPE ..STOP

BETA	MC	MCD	ALFA	PKAUG	BMIN	DXMIN	CONCAP	ENDCAP
0.100	0.005	0.	0.600	1.500	0.050	0.030	0.015	0.005

WE=0.004 STOP

ONE = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

$\rho_{CH} = 0.890 \quad 0.624 \quad 0.508 \quad 0.439 \quad 0.392 \quad 0.358 \quad 0.331 \quad 0.310 \quad 0.292$

30

PROPERTY CHANGES, DELTA E

BURN = 0.04777 EXP = 0.01456 SAFE = 0.02000 TOTAL = 0.08233

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.5890	0.4110	0.1644	0.2466

HEXAGONAL CORNER CORRECTION FACTOR = 1.1840

NUMBER OF HEAT PIPES = 115.0234

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 266

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 86.5

PIPE DELTA T ACROSS HEAT PIPE WALL = 14.5

AVERAGE FUEL TEMPERATURE = 1643.4

MAXIMUM FUEL TEMPERATURE = 1708.3

ELUEN FRACTION OF U235 = 0.0796

FISSION DENSITY (FISSIONS/CM\*\*3) = 8.598E+20

FUEL SHELLING: VOLUME % = 0.

REACTOR DIMENSIONS, METERS

FUEL ELEMENT DIMENSIONS, MM

0.4554	CORE DIAMETER	26.54	WIDTH ACROSS HEX FLATS
0.4554	CORE HEIGHT	27.87	EQUIV. FUEL ELEMENT DIA
0.6854	REACTOR DIAMETER	27.87	EQUIV. FUEL REGION O.D.
0.6854	REACTOR HEIGHT	17.87	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	13.84	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	150.41	VAPOR AREA: MM <sup>2</sup>
1.5504	TOTAL HEAT PIPE LENGTH		
1.6554	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTION WEIGHTS, KILOGRAMS

416.0 FUEL: 0235 N443 = 210.9

445.0 REFLECTOR

424.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 272.05

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 MG)

92.1 SUPPORT STRUCTURE (7% OF REACTOR WT)

1407.5 TOTAL REACTOR + HEAT PIPES

91.91	MM/M <sup>2</sup> ♦ 2. AVG POWR IN FUELSPACE	15.04	KW. POWER PER HEAT PIPE
99.99	MM/M <sup>2</sup> ♦ 2. HTPIPE AXIAL HT FLUX	0.760	MM/M <sup>2</sup> ♦ 2. HTPIPE RAD HTFLX

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TYPE 30 OF 100

PROG NO. 15 5-18-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
PR=0.2 THP=1750. STOP

0.200 (PR) REACTOR POWER:MW (KCORE) (1.2/UC:UO2) CORE #MO60UO2  
1750. (THP) HEAT PIPE TEMP:DEG K (KREF) (1.2/BE:BE0) REFLECTOR #BE0  
3650. (TIME) LIFETIME: DAYS (KHP) (1.2/3/NS:MO:M) HEAT PIPE #MO  
1.00 (ELD) CORE L/D RATIO (KVAPOR) (1.2/LI:NA) VAPOR #LI  
10.0 (CARL) AXIAL HT FLUX:KW/CM2 (IOPTN) (1.2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE : 1-CODE AT DESIGN: 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCORE(M) XREF(M) VNFT FBETA NPIPE ..STOP  
NPIPE=84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG BMIN DXMIN CORCAP ENDCAP  
0.100 0.004 0. 0.600 1.500 0.050 0.000 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. VCN=0, PKAVG=2, ETC ...STOP

VC=0.012 STOP

ELD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.207 0.142 0.115 0.099 0.088 0.081 0.074 0.070 0.066

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*  
GO

BETA = 0.1000 VNF = 0.1501 VF = 0.8499 DX = 0.1000 DC = 0.3115

REACTIVITY CHANGES: DELTA K

BURN = 0.00464 EXP = 0.01612 SAFE = 0.02000 TOTAL = 0.04076

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.9558 0.0442 0.0177 0.0265

HEXAGONAL CORNER CORRECTION FACTOR = 1.0020

NUMBER OF HEAT PIPES = 31.5836

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 75.2

Avg DELTA T ACROSS HEAT PIPE WALL = 3.4

AVERAGE FUEL TEMPERATURE = 1778.4

MAXIMUM FUEL TEMPERATURE = 1830.3

BURN FRACTION OF U235 = 0.0077

FISSION DENSITY (FISSIONS/CM\*\*3) = 8.345E+19

FUEL SWELLING: VOLUME % = 51.4

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REACTOR DIMENSIONS: METERS

0.3115 CORE DIAMETER

0.3115 CORE HEIGHT

0.5415 REACTOR DIAMETER

0.5215 REACTOR HEIGHT

0.1000 REFLECTOR THICKNESS

1.0000 PIPE LENGTH OUTSIDE REACTOR

1.4165 TOTAL HEAT PIPE LENGTH

1.5215 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS: MM

32.18 WIDTH ACROSS HEX FLATS

33.79 EQUIV. FUEL ELEMENT DIA

33.79 EQUIV. FUEL REGION O.D.

7.11 HEAT PIPE O.D.

5.51 VAPOR DIAMETER

23.80 VAPOR AREA: MM\*\*2

REACTOR HEIGHTS: KILOGRAMS

214.3 FUEL: U235 MASS = 108.7

240.2 REFLECTOR

19.3 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 13.60

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

35.5 SUPPORT STRUCTURE (7% OF REACTOR WT)

542.2 TOTAL REACTOR + HEAT PIPES

8.92 MW/M\*\*3: AVG POWR IN FUELSpace 2.38 KW/POWER PRP HEAT PIPE

100.04 MW/M\*\*2: HTPIPE AXIAL HT FLUX 0.442 MW/M\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

PROG NO. 16 5-18-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
PR=0.4 STOP

0.400 (PR) REACTOR POWER:MW (KCORR) (1,2/UC:UB2) CORE #MO60UB2  
1750. (THP) HEAT PIPE TEMP:DEG K (KREF) (1,2/BE:BE0) REFLECTOR #BRO  
3650. (TIME) LIFETIME:DAYE (KHP) (1,2,3/NS:MO:W) HEAT PIPE #MC  
1.00 (ELD) CORE L/D RATIO (K/VAPOR) (1,2/LI:NA) VAPOR #LI  
10.0 (OAXL) AXIAL HT FLUX:KW/CM2 (IDPTN) (1,2) OPTION #2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE: 1-CODE PT DESIGN; 2-SPECIFIED DESIGN  
TYPE IN ANY OF FOLLOWING: DCORE(M) KREF(M) UNFT FBETA NPIPE ..STOP  
NPIPE=84 STOP

84 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG BMIN DXMIN CORCOR ENDCOR  
0.100 0.012 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP

STOP

ELD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.293 0.201 0.163 0.140 0.125 0.114 0.105 0.098 0.093

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1000 UNF =0.1848 UF =0.8152 DX =0.1000 DC =0.3214

REACTIVITY CHANGES: DELTA K

BURN = 0.00981 EXP = 0.01612 SAFE = 0.02000 TOTAL = 0.04493

FUEL ELEMENT VOLUME FRACTIONS

CLADDING FUEL REGION HEAT PIPE WALL+WICK VAPOR  
0. 0.9167 0.0913 0.0333 0.0500

HEXAGONAL CORNER CORRECTION FACTOR =1.0070

NUMBER OF HEAT PIPES = 46.5570

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 84

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 110.8

AVG DELTA T ACROSS HEAT PIPE WALL = 6.5

AVERAGE FUEL TEMPERATURE =1793.5

MAXIMUM FUEL TEMPERATURE =1870.7

BURN FRACTION OF U235 =0.0147

FISSION DENSITY (FISSIONS/CM\*\*3) = 1.385E+20

FUEL SWELLING: VOLUME % = 2.6

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REACTOR DIMENSIONS: METERS

0.3214	CORE DIAMETER	33.19	WIDTH ACROSS HEX PLATS
0.3214	CORE HEIGHT	34.85	EQUIV. FUEL ELEMENT DIA
0.5514	REACTOR DIAMETER	34.85	EQUIV. FUEL REGION O.D.
0.5314	REACTOR HEIGHT	10.06	HEAT PIPE O.D.
0.1000	REFLECTOR THICKNESS	7.79	VAPOR DIAMETER
1.0000	PIPE LENGTH OUTSIDE REACTOR	47.66	VAPOR AREA: MM**2
1.4264	TOTAL HEAT PIPE LENGTH		
1.5314	OVERALL REACTOR+HEAT PIPE LENGTH		

REACTOR WEIGHTS: KILOGRAMS

225.6 FUEL: U235 MASS = 114.4  
251.9 REFLECTOR  
38.8 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 27.22  
33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)  
38.5 SUPPORT STRUCTURE (7% OF REACTOR WT)

587.9 TOTAL REACTOR + HEAT PIPES

16.94 MW/M\*\*3:AVG POWR IN FUELSpace 4.76 KW-POWER PER HEAT PIPE  
99.92 MW/M\*\*2:HTPIPE AXIAL HT FLUX 0.605 MW/M\*\*2:HTPIPE RAD HTFLX

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PROB NO. 17 5-18-78 TYPE NEW INPUT: PR=1. KNP=2 ...STOP  
PR=0.7 STOP

0.700 (PR) REACTOR POWER:MW (KCORE) (1,2/UC:UO2) CORE =MO60UO2  
1750. (THP) HEAT PIPE TEMP:DEG K (KREF) (1,2/BE:BE0) REFLECTOR =BE0  
3650. (TIME) LIFETIME: DAYS (KNP) (1,2,3/NS:MO:W) HEAT PIPE =MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI:NA) VAPOR =LI  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPTH) (1,2) OPTION =2  
200. (DTMAX) MAX FUEL DELTA T:DEG K  
1.00 (WPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE: 1-CODE PT DESIGN: 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: 1-CORE(M) XREF(M) UNFT P/BETA NPIPE ...STOP  
NPIPE=120 STOP

120 (NPIPE) NO. OF HEAT PIPES

BETA UC VCD ALFA PKAVG BMIN DXMIN CURGAP ENDGAP  
0.100 0.012 0. 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP: OR NEW CONSTANTS IE. UC=0. PKAVG=2. ETC ...STOP

UC=0.008 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.380 0.263 0.214 0.184 0.165 0.150 0.139 0.130 0.122

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA =0.1000 UNF =0.2264 VF =0.7736 DX =0.1000 DC =0.3348

REACTIVITY CHANGES: DELTA K

BURN = 0.01435 EXP = 0.01612 SAFE = 0.02000 TOTAL = 0.05047

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.8665	0.1335	0.0534	0.0801

HEXAGONAL CORNER CORRECTION FACTOR =1.0180

NUMBER OF HEAT PIPES = 61.1476

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 120

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 101.9

AVG DELTA T ACROSS HEAT PIPE WALL = 7.7

AVERAGE FUEL TEMPERATURE =1791.7

MAXIMUM FUEL TEMPERATURE =1863.4

BURN FRACTION OF U235 =0.0239

FISSION DENSITY (FISSIONS/CM\*\*3) = 2.584E+20

FUEL SWELLING: VOLUME % = 4.3

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REACTOR DIMENSIONS: METERS

REACTOR DIMENSIONS: METERS	FUEL ELEMENT DIMENSIONS: MM
0.3348 CORE DIAMETER	29.00 WIDTH ACROSS HEX FLATS
0.3348 CORE HEIGHT	30.44 EQUIV. FUEL ELEMENT DIA
0.5648 REACTOR DIAMETER	30.44 EQUIV. FUEL REGION O.D.
0.5448 REACTOR HEIGHT	11.12 HEAT PIPE O.D.
0.1000 REFLECTOR THICKNESS	8.62 VAPOR DIAMETER
1.0000 PIPE LENGTH OUTSIDE REACTOR	58.31 VAPOR AREA: MM**2
1.4398 TOTAL HEAT PIPE LENGTH	
1.5448 OVERALL REACTOR+HEAT PIPE LENGTH	

REACTOR WEIGHTS: KILOGRAMS

242.3 FUEL: U235 MASS = 122.8
268.5 REFLECTOR
68.5 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 47.58
33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)
42.9 SUPPORT STRUCTURE (7% OF REACTOR WT)

655.2 TOTAL REACTOR + HEAT PIPES

27.62 MW/M\*\*3: AVG POWR IN FUELSpace 5.83 KW: POWER PER HEAT PIPE  
100.05 MW/M\*\*2: HTPIPE AXIAL HT FLUX 0.644 MW/M\*\*2: HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

\*\*\*\*\*

PROG NO. 18 5-18-78 TYPE NEW INPUT: PR=1. KHP=2 ...STOP  
PR=1. STOP

1.000 (PR) REACTOR POWER:MW (KCORE) (1,2/UC,UO2) CORE =NO60UO2  
1750. (THP) HEAT PIPE TEMP:DEG K (KREF) (1,2/BE,BEO) REFLECTOR =BEO  
3650. (TIME) LIFETIME: DAYS (KHP) (1,2,3/ND,MO,W) HEAT PIPE =MO  
1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI,NA) VAPOR =LI  
10.0 (DAXL) AXIAL HT FLUX:KW/CM2 (IOPN) (1,2) OPTION =2  
200. (DTFMAX) MAX FUEL DELTA T:DEG K  
1.00 (HPL1) PIPE EXTENSION:M

NOTE: OPTIONS ARE: 1-CODE AT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING: DCORE(1) XREF(M) UNFT FBETA NPIPE ...STOP  
NPIPE=162 STOP

162 (NPIPE) NO. OF HEAT PIPES

BETA	VC	UCD	ALFA	PKAVG	BMIN	DXMIN	CORCAP	ENDCAP
0.100	0.008	0.	0.600	1.500	0.050	0.080	0.015	0.005

TYPE: STOP: OR NEW CONSTAN: IE. VC=0. PKAVG=2. ETC ...STOP

VC=0.006 STOP

SLD INDEX = 3

UNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 \*

DCH = 0. 0.449 0.313 0.255 0.220 0.197 0.179 0.166 0.155 0.146

\*\*\*\*\* TYPE GO OR START OVER \*\*\*\*\*

GO

BETA = 0.1000 UNF = 0.2630 UF = 0.7370 DX = 0.1000 DC = 0.3482

REACTIVITY CHANGES: DELTA K

BURN = 0.01915 EXP = 0.01612 SAFE = 0.02000 TOTAL = 0.05527

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.8238	0.1762	0.0705	0.1057

HEXAGONAL CORNER CORRECTION FACTOR = 1.0315

NUMBER OF HEAT PIPES = 71.2219

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 162

TEMPERATURE SUMMARY: DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.9

AVG DELTA T ACROSS HEAT PIPE WALL = 7.8

AVERAGE FUEL TEMPERATURE = 1787.1

MAXIMUM FUEL TEMPERATURE = 1849.7

BURN FRACTION OF U235 = 0.0319

FISSION DENSITY (FISSIONS/CM\*\*3) = 3.447E+20

FUEL SWELLING: VOLUME % = ~~4~~ ~ 5.7

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REACTOR DIMENSIONS: METERS

REACTOR DIMENSIONS: METERS	FUEL ELEMENT DIMENSIONS: MM
0.3482 CORE DIAMETER	25.97 WIDTH ACROSS HEX FLATS
0.3482 CORE HEIGHT	27.27 EQUIV. FUEL ELEMENT DIA
0.5782 REACTOR DIAMETER	27.27 EQUIV. FUEL REGION O.D.
0.5582 REACTOR HEIGHT	11.45 HEAT PIPE O.D.
0.1000 REFLECTOR THICKNESS	8.87 VAPOR DIAMETER
1.0000 PIPE LENGTH OUTSIDE REACTOR	61.74 VAPOR AREA: MM**2
1.4532 TOTAL HEAT PIPE LENGTH	
1.5582 OVERALL REACTOR+HEAT PIPE LENGTH	

REACTOR WEIGHTS: KILOGRAMS

259.4 FUEL: U235 MASS = 131.5

285.5 REFLECTOR

98.8 HEAT PIPES: WT/UNIT LENGTH (KG/M) = 68.02

33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)

47.4 SUPPORT STRUCTURE (7% OF REACTOR WT)

724.1 TOTAL REACTOR + HEAT PIPES

36.84 MW/M\*\*3 AVG POWR IN FUELSpace 6.17 KW/POWER PER HEAT PIPE

99.98 MW/M\*\*2 HTPIPE AXIAL HT FLUX 0.637 MW/M\*\*2 HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR STOP

V PROB NO. 1 6-21-78 TYPE NEW INPUT: PR=1, KHP=2 ...STOP  
 PR=2: THP=1750. TIME=3650. KCORE=5 KREF=2 KVAPOR=1 IOPTN=2 STOP

2.000 (PR) REACTOR POWER;MW (KCORE) (1,2/UC,UO2) CORE =MO60UO2  
 1750. (THP) HEAT PIPE TEMP;DEG K (KREF) (1,2/BE,BEO) REFLECTOR =BEO  
 3650. (TIME) LIFETIME;DAYS (KHP) (1,2/3/NS;MO;W) HEAT PIPE =MO  
 1.00 (SLD) CORE L/D RATIO (KVAPOR) (1,2/LI;NA) VAPOR =LI  
 10.0 (DAXL) AXIAL HT FLUX;KW/CM2 (IOPTN) (1,2) OPTION =2  
 200. (DTFMAX) MAX FUEL DELTA T;DEG K  
 1.00 (HPL1) PIPE EXTENSION;M

NOTE: OPTIONS ARE : 1-CODE PT DESIGN; 2-SPECIFIED DESIGN

TYPE IN ANY OF FOLLOWING : DCORE(M) XREF(M) VNFT FBST; NPIPE ..STOP  
 NPIPE=210 STOP

210 (NPIPE) NO. OF HEAT PIPES

ZETA VC VCD ALFA PKAVG BMIN DKMIN CORGAP ENDGAP  
 0.100 0. 0.050 0.600 1.500 0.050 0.080 0.015 0.005

TYPE: STOP; OR NEW CONSTANTS IE. VC=0. PKAVG=2. ETC ...STOP

VC=0.005 VCD=0. STOP

SLD INDEX = 3

VNF = 0.100 0.200 0.300 0.400 0.500 0.600 0.700 0.800 0.900 1.000

DC = 0.299 0.326 0.363 0.412 0.475 0.562 0.694 0.961 1.754 +.

DCH = 0. 0.632 0.442 0.360 0.311 0.278 0.253 0.234 0.219 0.207

♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ TYPE GO OR START OVER ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦ ♦

GO...ADJUST ZETA FOR 3.33% SWELLING

ZETA =0.1000 VNF =0.3566 VF =0.6434 DX =0.1000 DC =0.3893

REACTIVITY CHANGES: DELTA K

BURN = 0.03138 EXP = 0.01612 SAFE = 0.02000 TOTAL = 0.06750

FUEL ELEMENT VOLUME FRACTIONS

CLADDING	FUEL REGION	HEAT PIPE	WALL+WICK	VAPOR
0.	0.7185	0.2815	0.1126	0.1689

HEXAGONAL CORNER CORRECTION FACTOR =1.0825

NUMBER OF HEAT PIPES = 92.2398

MINIMUM PRACTICAL (OR SPECIFIED) NUMBER OF HEAT PIPES = 210

TEMPERATURE SUMMARY;DEGREE KELVIN

MAXIMUM FUEL DELTA T = 87.8

AVG DELTA T ACROSS HEAT PIPE WALL = 10.8

AVERAGE FUEL TEMPERATURE =1790.1

MAXIMUM FUEL TEMPERATURE =1854.0

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BURN FRACTION OF U235 =0.0523

FISSION DENSITY (FISSIONS/CM\*\*3) = 5.648E+20

FUEL SWELLING;VOLUME % = 3.10 x3 =9.3

REACTOR DIMENSIONS; METERS

0.3893 CORE DIAMETER  
 0.3893 CORE HEIGHT  
 0.6193 REACTOR DIAMETER  
 0.5993 REACTOR HEIGHT  
 0.1000 REFLECTOR THICKNESS  
 1.0000 PIPE LENGTH OUTSIDE REACTOR  
 1.4943 TOTAL HEAT PIPE LENGTH  
 1.5993 OVERALL REACTOR+HEAT PIPE LENGTH

FUEL ELEMENT DIMENSIONS; MM

25.52 WIDTH ACROSS HEX FLATS  
 26.80 EQUIV. FUEL ELEMENT DIA  
 26.80 EQUIV. FUEL REGION O.D.  
 14.22 HEAT PIPE O.D.  
 11.01 VAPOR DIAMETER  
 95.26 VAPOR AREA; MM\*\*2

REACTOR WEIGHTS; KILOGRAMS

316.6 FUEL; U235 MASS = 160.5  
 341.3 REFLECTOR  
 203.3 HEAT PIPES; WT/UNIT LENGTH (KG/M) = 136.03  
 33.0 CONTROL SYSTEM (ASSUME CONSTANT = 33 KG)  
 62.6 SUPPORT STRUCTURE (7% OF REACTOR WT)

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 956.7 TOTAL REACTOR + HEAT PIPES

60.37 MW/M\*\*3;AVG POWR IN FUELSPACE

9.52 KW;POWER PER HEAT PIPE

99.98 MW/M\*\*2;HTPIPE AXIAL HT FLUX

0.707 MW/M\*\*2;HTPIPE RAD HTFLX

\*\*\*\*\*

TYPE GO OR END

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